

JAN 15 1982

Docket No.: STN 50-483

Mr. D. F. Schnell
Vice President - Nuclear
Union Electric Company
P. O. Box 149
St. Louis, Missouri 63166

Dear Mr. Schnell:

Subject: Request for Additional Information for the Review of the
Callaway Plant, Unit 1 Regarding Reactor Systems

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As a result of our continuing review of the Callaway Plant, Unit 1 FSAR, we find that we need additional information to complete our evaluation. The specific information required is in the area of reactor systems and is presented in the Enclosure.

To maintain our licensing review schedule, we will need responses to the enclosed request by February 4, 1982. If you cannot meet this date, please inform us within seven days after receipt of this letter of the date you plan to submit your responses so that we may review our schedule for any necessary changes. Please note that an advance copy of this question was provided to SNUPPS on December 21, 1981.

Please contact Dr. G. E. Edison, Callaway Licensing Project Manager, if you desire any discussion or clarification of the enclosed request.

Sincerely,

Original signed by
W. J. Youngblood

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing



Enclosure:
As stated

cc w/encl.: See next page

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"The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511."

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DATE	1/15/82	1/15/82					

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REQUEST FOR ADDITIONAL INFORMATION

440.0C Reactor Systems Branch

440.1C The analyses of a locked reactor coolant pump rotor and a sheared reactor coolant pump shaft in the FSAR assumes the availability of offsite power throughout the event. In accordance with Standard Review Plan 15.3.3 and GDC 17, we require that this event be analyzed assuming turbine trip and coincident loss of offsite power to the undamaged pumps.

Appropriate delay times may be assumed for loss of offsite power if suitably justified.

Steam generator tube leakage should be assumed at the rates specified in the Technical Specifications.

The event should also be analyzed assuming the worst single failure of a safety-system active component. Maximum technical specification primary system activity and steam generator tube leakage should be assumed. The analyses should demonstrate that offsite doses are less than 10 CFR 100 guidelines values.