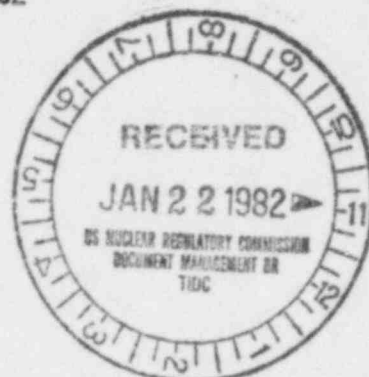


Docket Nos. 50-220

JAN 18 1982

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Siebert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202



Dear Mr. Dise:

Ref: Status of NUREG-0737 Items II.F.1.1 and II.F.1.2

NUREG-0737 Items II.F.1.1 and II.F.1.2 require the installation of high range noble gas effluent monitors and provisions for effluent monitoring of radioiodines at accident conditions, respectively, by January 1, 1982. Since a postimplementation review is planned, we are presently reviewing only deviations to the stated NRC positions.

Based on our review of your submittals and discussions with your instrument vendor, no technical deviations from our stated positions are requested to implement NUREG-0737 Items II.F.1.1 and II.F.1.2. If this understanding is not correct, we request formal notification of such within 30 days of receipt of this letter.

Your submittal further indicated that you anticipate problems meeting the implementation date of January 1, 1982 for Item II.F.1.1 and II.F.1.2 and that the implementation date for final modification will be delayed until January 1, 1983. The Commission will be reviewing all requests for delayed implementation of all NUREG-0737 items and we will inform you of their decision for any schedule revisions.

Changes to your plant Technical Specifications (TS) for Items II.F.1.1 and II.F.1.2 were envisioned for NUREG-0737 implementation. Sample standard TS pages are provided as Enclosure 1 for your assistance. The schedule for requesting such TS changes will be determined following our receipt of the OMB clearance of all NUREG-0737 items currently under review pursuant to the Paperwork Reduction Act of 1980.

In these sample Technical Specifications the action required based on the inoperative status of the accident monitoring instrumentation identified in the Table (3.3.7.5-1) has intentionally been left blank. Upon receipt of OMB clearance it is requested that you propose the action required for your TS changes related to NUREG-0737 Items II.F.1.1 and II.F.1.2. Justification or bases for your proposed action should be included. An acceptable action based on the inoperable status of the noble gas effluent monitors might be:

OFFICE	8202080424 820118 PDR AL-DCK 05000220 PDR					
SURNAME	P					
DATE						

With the required monitors inoperable, either restore the inoperable Channel(s) to OPERABLE status within 72 hours, or:

1. Initiate the preplanned alternate method of monitoring the appropriate parameter(s) and
2. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

If you have questions on any portions of this letter, please contact your assigned NRC project manager.

Sincerely,

Original signed by
D. B. Vassallo

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosure:
Sample TS Pages

cc: w/enclosure:
See next page

Distribution:
 Docket File
 NRC PDR
 Local PDR
 TERA
 NSIC
 ORB#2 Reading
 DEisenhut
 P. Polk
 S. Norris
 M. Conner
 P. Stoddart
 OELD
 AEOD
 IE-3
 ACRS-10

OFFICE	ORB#2	ORB#2	ORB#3	ORB#2		
SURNAME	SNorris	PPolk:pbe	MConner	Tippolito		
DATE	1/15/82	1/15/82	1/15/82	1/18/82		

Mr. Donald P. Dise

cc:

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LeBoeuf, Lamb, Leiby & MacRae
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Suite 1100
Washington, D. C. 20036

State University College at Oswego
Penfield Library - Documents
Oswego, New York 13126

Resident Inspector
c/o U.S. NRC
P.O. Box 126
Lycoming, New York 13093

Carl D. Hobelman, Esq.
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Suite 1100
Washington, D.C. 20036

Ronald C. Haynes
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

INSTRUMENTATIONACCIDENT MONITORING INSTRUMENTATIONLIMITING CONDITION FOR OPERATION

3.3.7.5 The accident monitoring instrumentation channels shown in Table 3.3.7.5-1 shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within days
or
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within hours
or

SURVEILLANCE REQUIREMENTS

4.3.7.5 Each of the above required accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.7.5-1.

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure	2	1
2. Reactor Vessel Water Level	2	1
3. Suppression Pool Water Level	2	1
4. Suppression Pool Water Temperature	6, 1/sector	6, 1/sector
5. Drywell/Containment Differential Pressure	2	1
6. Drywell Pressure	2	1
7. Drywell and Control Rod Drive Cavity Temperature	2	1
8. Containment Hydrogen Concentration Analyzer and Monitor	2	1
9. Drywell Hydrogen Concentration Analyzer and Monitor	2	1
10. Containment Pressure	2	1
11. Containment Air Temperature	2	1
12. Safety/Relief Valve Tail Pipe Pressure Switch Position Indicators	1/valve 2 [#]	1/valve 1 [#]
13. Containment/Drywell Area Monitors	1	1
14. Containment Ventilation Monitor	1	1
15. Off-gas and Radwaste Bldg. Ventilation Monitor	1	1
16. Fuel Handling Area Ventilation Monitor	1	1
17. Turbine Bldg. Ventilation Monitor	1	1
18. Standby Gas Treatment System A & B Exhaust Monitors	1/each	1/each

[#] Each for containment and drywell.

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	R
2. Reactor Vessel Water Level	M	R
3. Suppression Pool Water Level	M	R
4. Suppression Pool Water Temperature	M	R
5. Drywell/Containment Differential Pressure	M	R
6. Drywell Pressure	M	R
7. Drywell and Control Rod Cavity Temperature	M	R
8. Containment Hydrogen Concentration Analyzer and Monitor	NA	Q*
9. Drywell Hydrogen Concentration Analyzer and Monitor	NA	Q*
10. Containment Pressure	M	R
11. Containment Air Temperature	M	R
12. Safety/Relief Valve Tail Pipe Pressure Switch Position Indicators	M	R
13. Containment/Drywell Area Monitors	M	R
14. Containment Ventilation Monitor	M	R
15. Off-gas and Radwaste Bldg. Ventilation Monitor	M	R
16. Fuel Handling Area Ventilation Monitor	M	R
17. Turbine Bldg. Ventilation Monitor	M	R
18. Standby Gas Treatment System, A & B Exhaust Monitors	M	R

*Using sample gas containing:

- a. One volume percent hydrogen, remainder nitrogen.
- b. Four volume percent hydrogen, remainder nitrogen.

INSTRUMENTATION

BASES

3/4.3.7.5 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess important variables following an accident. (This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations").

3/4.3.7.6 SOURCE RANGE MONITORS

The source range monitors provide the operator with information of the status of the neutron level in the core at very low power levels during startup and shutdown. At these power levels, reactivity additions should not be made without this flux level information available to the operator. When the intermediate range monitors are on scale adequate information is available without the SRMs and they can be retracted.

3/4.3.7.7 TRAVERSING IN-CORE PROBE SYSTEM

The OPERABILITY of the traversing in-core probe system with the specified minimum complement of equipment ensures that the measurements obtained from use of this equipment accurately represent the spatial neutron flux distribution of the reactor core.

3/4.3.7.8 CHLORINE (AND AMMONIA) DETECTION SYSTEM (Optional)

The OPERABILITY of the chlorine (and ammonia) detection system ensures that an accidental chlorine (and/or ammonia) release will be detected promptly and the necessary protective actions will be automatically initiated to provide protection for control room personnel. Upon detection of a high concentration of chlorine (and/or ammonia), the control room emergency ventilation system will automatically be placed in the (isolation) mode of operation to provide the required protection. (The detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.95 "Protection of Nuclear Power Plant Control Room Operators against an Accidental Chlorine Release", February 1975.)