OPERATING DATA REPORT

DOCKET NO. 50-266

DATE January 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
2. REPORTING PERIOD: DECEMBER 1981
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.

NOTES

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
   NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): 445.0
10. REASONS FOR RESTRICTIONS, (IF ANY): Maximum dependable capacity reduced because of self-imposed hot leg temperature limitation in an attempt to limit steam generator tube corrosion.

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	8,760	97,776
12. NUMBER OF HOURS REACTOR WAS CRITICAL	514.1	6,882.6	80,317.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.3	1.6	607.3
14. HOURS GENERATOR ON LINE	472.4	6,811.2	77,948.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	764.3
16. GROSS THERMAL ENERGY GENERATED (MWH)	601,995	8,514,396	107,352,840
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	197,800	2,761,560	36,021,280
18. NET ELECTRICAL ENERGY GENERATED (MWH)	185,779	2,613,925	34,281,211
19. UNIT SERVICE FACTOR	63.5	77.8	79.7
20. UNIT AVAILABILITY FACTOR	63.5	77.8	80.5
21. UNIT CAPACITY FACTOR (USING MDC NET)	50.4	60.3	71.9
22. UNIT CAPACITY FACTOR (USING DER NET)	50.2	60.0	70.5
23. UNIT FORCED OUTAGE RATE	0.0	0.2	3.0

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
    NONE

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE January 6, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH December, 1981

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	- 2	11	22	21	415
2	- 1	12	97	22	416
3	- 6	13	319	23	417
4	- 9	14	351	24	419
5	-10	15	414	25	393
6	- 8	16	392	26	403
7	-11	17	391	27	417
8	-13	18	416	28	417
9	-13	19	414	29	416
10	33	20	415	30	418
				31	418

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(1-77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December, 1981

DOCKET NO. 50-266
UNIT NAME Point Beach Unit 1
DATE January 6, 1982
COMPLETED BY C. W. Fay
TELEPHONE 414/277-2811

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action To Prevent Recurrence
5	811009	S	1501.5	C	1	N/A	ZZ	ZZZZZZ	Returned to power following completion of refueling outage.

[1] F: Forced
S: Scheduled

[2] Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

[3] Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

[4] Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

[5] Exhibit I- Same Source

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(01-78)

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date January 6, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Unit 1 remained in shut-down condition as the ninth refueling outage progressed. Preparations for initial criticality were interrupted at 1833 hours on December 2, 1981 when the supply breaker from 1A03 and 1A05 opened due to undervoltage caused by the voltage dip during a reactor coolant pump start-up. These TMI backfit relays were set with a time delay of 13.5 seconds at 90% voltage (Technical Specification number), however, the pump start time to bus voltage recovery is about 18 seconds and thus these relays timed out and caused the trip of 1A52-57 breaker. During this event, the 3D diesel generator started and assumed the 1A05 loads. At 1922 hours a call was made to the NRC via the red phone. The situation was returned to normal by 1932 hours. The time delay on these relays was increased to 20 seconds and successful pump start at 1935 hours verified these settings. This reset was given NRC concurrence to be set at any value less than 60 seconds on December 3. Progress to start-up continued with the setting of containment integrity on December 4 at 0702 hours and the commencement of steam generator crevice cleaning at 0720 hours on the same day. Cold rod drop tests were completed at 0300 hours on December 5; the steam generator crevice flush was completed at 1145 hours; and the heat-up for the ASME Section XI ten-year hydrostatic test of Class 1 components was begun. This hydrostatic test was satisfactorily completed at 0605 hours on December 6. On December 8 hot valve line-ups were completed at 1158 hours with hot rod drop tests completed at 1927 hours the same day. The Unit 1 reactor was critical at 0101 hours on December 9 concluding the refueling outage.

Unit 1 was brought to 432 MWe net on December 14 after all outage-associated testing had been completed. Two trips were experienced on December 9 due to a spike and subsequent failure of the channel 31B source range detector. This occurred while the reactor was subcritical. The NRC was notified of the event as it was red phone reportable. Another unit trip from 80 MWe occurred on December 11 due to failure to block the low power range trip; again, the NRC was notified via the red phone. Problems with the electro-hydraulic system governor valves caused a turbine trip on December 11 from low power conditions. The auxiliary governor solenoid valves were replaced later the same day.

Unit 1 operated at 420 MWe net throughout the remainder of the period with three load reductions. Load was reduced to 138 MWe net at 2056 hours on December 16 for condenser tube repairs. Load was reduced to approximately 330 MWe net for six hours on December 25 and 26 at the request of the Power System supervisor.

On December 10 following rephasing of the unit to the line at the end of the refueling outage, both steam flow channels for loop "A" were discovered to be indicating zero flow.

The flow indications from FC-464 and FC-465 were noted at zero flow at approximately 0800 hours on December 10. It is not unusual for flow indications to fluctuate when the unit is first placed on line and the zero flow condition could not be detected until the unit had reached approximately 15% power.

Following investigations into the problem, flow channel FC-464 was placed in the trip position at 0830 hours. This met the Technical Specification for limiting conditions of operation since the logic is one out of two per loop for steam flow. With one steam flow channel in trip for the "A" loop, the high and high-high steam flow portion of the logic is already in for steam line isolation and reactor trip. Steam line isolation is initiated by a high steam flow coincident with low Tave and safety injection or a high-high steam flow coincident with safety injection. A reactor trip is initiated by a steam flow/feed flow mismatch (steam flow higher than feed flow) and a low steam generator level. Following further review and discussions into the problem, the other flow channel, FC-465, was placed in the trip position at 1030 hours on December 10. Placing the remaining channel in trip satisfied the Technical Specifications in providing the minimum degree of redundancy.

The zero flow indications were a result of improper zero settings on both transmitters. These transmitters were installed during the outage as part of TMI-required modifications and were checked out and aligned satisfactorily on November 7. Although the exact cause of the transmitter misalignment cannot be determined, the cause is believed to be related to contractor activity in the area; either by movement of equipment in the area or by an inadvertent adjustment of the exposed zero adjust screw by an unauthorized person.

Both transmitters were realigned and put back into service by 1230 hours on December 10.

This event is reportable in accordance with Technical Specification 15.6.9.2.A.2.

On December 19 at 2325 hours, Unit 1 "A" steam generator pressure indicator 1PI-482 was observed not to be indicating properly. It was discovered that the sensing lines to pressure transmitter 1PT-482 had frozen up. These sensing lines were recently rearranged as part of post-TMI backfit modifications. Further investigation revealed holes in the sensing line insulation allowed cold air to infiltrate the insulation and permit the water in the sensing lines to freeze. The sensing lines were thawed and a second heat tracing circuit for the affected sensing lines was activated. This occurrence is reportable as a 30-day report in accordance with Technical Specification 15.6.9.2.B.2, "Conditions leading to operation in a degraded mode permitted by a limiting condition for operation", (Licensee Event Report No. 81-020/03L-0).

Safety-related maintenance during this period included continued efforts in satisfying requirements for block walls, seismic restraints, and TMI electrical upgraded equipment installation; adjustment of the undervoltage relays serving the A05 and A06 buses; replacement of several failed relays; completion of replacement and repair of the reactor coolant sample valves; and correction of problems identified during the Class 1 hydrostatic test.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE January 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2 . NOTES .
2. REPORTING PERIOD: DECEMBER 1981 . .
3. LICENSED THERMAL POWER (MWT): 1518. . .
4. NAMEPLATE RATING (GROSS MWE): 523.8 . .
5. DESIGN ELECTRICAL RATING (NET MWE): 497. . .
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 5[illegible]. . .
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495. . . . . .
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	8,760	82,561
12. NUMBER OF HOURS REACTOR WAS CRITICAL	736.9	7,840.4	74,390.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	6.7	193.0
14. HOURS GENERATOR ON LINE	724.9	7,760.1	73,058.6
15. UNIT RESERVE SHUTDOWN HOURS	0.0	54.1	178.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,073,535	11,467,098	100,256,844
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	365,310	3,899,160	34,023,010
18. NET ELECTRICAL ENERGY GENERATED (MWH)	345,456	3,720,045	32,383,466
19. UNIT SERVICE FACTOR	97.4	88.6	88.5
20. UNIT AVAILABILITY FACTOR	97.4	89.2	88.7
21. UNIT CAPACITY FACTOR (USING MDC NET)	94.9	85.8	79.9
22. UNIT CAPACITY FACTOR (USING DER NET)	94.5	85.4	78.9
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.7

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling outage scheduled for April 16, 1982, expected to last approximately 14 weeks.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-301

UNIT NAME Point Beach Unit 2

DATE January 6, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH December, 1981

DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	486	11	489	21	492
2	488	12	488	22	492
3	488	13	488	23	495
4	489	14	482	24	494
5	488	15	492	25	490
6	497	16	488	26	446
7	481	17	487	27	456
8	477	18	475	28	457
9	488	19	33	29	490
10	488	20	489	30	490
				31	480

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(1-77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December, 1981

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE January 6, 1982
COMPLETED BY C. W. Fay
TELEPHONE 414/277-2811

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action To Prevent Recurrence
6	811219	S	19.1	B	1	N/A	ZZ	ZZZZZZ	The unit was taken off line for turbine stop valve and moisture separator reheater maintenance.

[1] F: Forced
S: Scheduled

[2] Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

[3] Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

[4] Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

[5] Exhibit I- Same Source

AD-28B
(01-78)

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No.	50-301
Unit Name	Point Beach Unit 2
Date	January 6, 1982
Completed By	C. W. Fay
Telephone	414/277-2811

Unit 2 operated at 490 MWe net throughout the period with one steam plant shut-down for turbine stop valve and moisture separator reheater maintenance. Unit 2 was off line at 0151 hours on December 19. Following secondary plant maintenance, Unit 2 was placed on line at 2057 hours the same day and was at 490 MWe at 0050 hours on December 20.

Training start-ups were conducted during the maintenance outage.

Safety-related maintenance during the period included repacking of the 2P2A charging pump.