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Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
Monthly Operating Report

United States Nuclear Regulatory Commission
Director, Office of Management Information & Program Control
Washington, D. C. 20555

Gentlemen:

In accordance with Appendix A, Technical Specifications, the Monthly Operating Report is submitted for the month of February, 1981.

Very truly yours,

C. N. Dunn
Vice President, Operations

Enclosures

cc: NRC Regional Office, King of Prussia, PA
P. Higgins, Prime Movers Committee

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DUQUESNE LIGHT COMPANY
Beaver Valley Power Station

NARRATIVE SUMMARY OF MONTHLY OPERATING EXPERIENCE - JANUARY 1981

January 1	Station in Operational Mode 4 at 950 PSIG and 337F. Heat-up of the reactor coolant system was in progress. Entered Operational Mode 3 at 0152 hours.
January 2 through January 5	Station in Operational Mode 3 at 200 PSIG and 541F. Heat-up of the reactor coolant system was in progress and the prerequisites for reactor criticality were being completed. At 2000 hours, a small packing leak developed on the 1B loop bypass flow isolation valve [1RC-45]. This leak could not be stopped. At 0303 hours on January 3, a station cooldown to Operational Mode 5 was begun. The Station entered Operational Mode 4 at 0940 hours and Operational Mode 5 at 1535 hours. Containment vacuum was broken at 1630 hours and RCS depressurization was begun at 2010 hours. The packing leak on [1RC-45] was repaired and the packing on other similar valves were checked and adjusted as necessary by 1220 hours on January 4. Filling the pressurizer was begun immediately and a steam bubble was established by 0300 hours on January 5. Drawing the containment vacuum was begun at 0550 hours.
January 6	Station in Operational Mode 5 at 317 PSIG and 153F. Heat-up of the reactor coolant system was in progress. At 0255 hours, with the RCS temperature at 180F, further heat-up was delayed to correct reactor coolant pump seal problems. Heat-up was resumed at 0650 hours and the Station entered Operational Mode 4 at 1213 hours. At 1555 hours, the Company Engineering Department informed the Station of a pipe-whip-restraint deficiency at the main steam line non-return valves and an RCS cooldown to Operational Mode 5 was begun to repair the restraints.
January 7	Station in Operational Mode 4 at 390 PSIG and 300F. RCS pressure was being maintained at less than 400 PSIG and RCS temperature was being maintained between 300F and 325F during main steam line restraint repairs. At 0745 hours, the Residual Heat Removal system was isolated and an increase of RCS pressure to 900 PSIG was commenced.
January 8	Station in Operational Mode 4 at 870 PSIG and 323F. At 1040 hours the 1A main feedwater stop-check isolation valve [MOV-FW-156A] was declared inoperable because the gear drive housing was discovered cracked. At 1850 hours, both Low Head Safety Injection (LHSI) Pumps were declared inoperable when they did not meet the Operational Surveillance Test (OST) acceptance criteria. The recirculation flow test line was discovered to have frozen.

NARRATIVE SUMMARY OF MONTHLY OPERATING EXPERIENCE - JANUARY 1981 (continued)

January 9 through January 10	Station in Operational Mode 4 at 920 PSIG and 321F. RCS was being maintained in the hot shutdown condition pending repairs to [MOV-FW-156A] and adding insulation to the LHSI Pump test line. At 1445 hours the LHSI pumps were declared operable after satisfactory completion of the surveillance tests, At 2100 hours, [MOV-FW-156A] was operable and RCS heat-up was commenced at 2127 hours. The Station entered Operational Mode 3 at 2320 hours with the RCS heat-up in progress.
January 11 through January 12	Station in Operational Mode 3 at 2235 PSIG and 545F. The reactor start-up was commenced at 1441 hours. The reactor was critical at 1535 hours and the main unit was synchronized at 0147 hours on January 12.
January 13	Station in Operational Mode 1 at nominal 88% reactor power.
January 14 through January 20	Station in Operational Mode 1 at nominal 99% reactor power.
January 21	Station in Operational Mode 1 at nominal 98% reactor power. Power level was reduced to 95% because of high backpressure in the main condenser. Electric power output varied slightly with main condenser performance.
January 22	Station in Operational Mode 1 at nominal 97% reactor power. An Emergency Preparedness Plan alert was declared at 0430 hours due to a high-high radiation alarm on the leak collection areas exhaust gas radiation monitor [RM-VS-105], which automatically diverts the discharge through the main filter banks. The maximum off-site gaseous releases did not exceed 3% MPC. The alert was terminated at 0848 hours. The apparent cause was gas accumulation in the pipe trenches being released into the main building space when the trench cover blocks were removed
January 23	Station in Operational Mode 1 at nominal 98% reactor power.
January 24 through January 26	Station in Operational Mode 1 at nominal 99% reactor power. The power level was reduced to 64% at 2050 hours to remove the main condenser section A from service for a tube cleanliness inspection. Condenser section A was returned to service at 0830 hours on January 25 and section B was removed from service for inspection when the circulating water outlet isolation valve for section B could not be opened. The power level was further reduced to 46%

at 1615 hours on January 26 in order to shut down the 1C Cooling Tower Pump to reduce the pressure on the valve. Power level was increased to 74% at 1840 hours with two (1A and 1D) cooling tower pumps in service. The 1C Cooling Tower Pump was placed into service at 2023 hours and the power level was increased to 93% by 2220 hours. Power level was reduced to 90% at 2232 hours due to high condensate temperatures.

January 27 through January 29	Station in Operational Mode 1 at nominal 90% reactor power. Maintaining the condenser vacuum/turbine backpressure was the major operational difficulty and some adjustment of power level was required to retain proper station operating conditions.
January 30 through January 31	Station in Operational Mode 1 at nominal 94% reactor power. At 2043 hours, commenced reducing power level to 59% to perform main condenser inspection and maintenance.

MAJOR SAFETY-RELATED MAINTENANCE - JANUARY 1981

1. The gear housing for the motor-operated 1A main feedwater isolation valve [MOV-FW-156A] was replaced after the housing was found to be cracked.