



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

January 11, 1982

SNRC-657

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Shoreham Nuclear Power Station - Unit 1
Docket No. 50-322



Dear Mr. Denton:

Enclosed herewith are sixty (60) copies of information which is either in response to specific NRC open items or confirmatory relative to past open items which are now closed. The scope of this specific submittal is tabulated on Attachment A.

We trust that the enclosed information is satisfactory. Should you have any questions or require additional information, please do not hesitate to contact this office.

Very truly yours,

J. L. Smith
Manager, Special Projects
Shoreham Nuclear Power Station

RWG:mp

Enclosure

cc: J. Higgins

*Boo
S/1/60*

8201130193 820111
PDR ADOCK 05000322
A PDR

Attachment A

I. Items submitted in response to specific NRC open items:

1. Item II.B.3 - Post Accident Sampling -
Procedures for the Determination of the Extent
of Core Damage
2. Item II.E.4.2 - Containment Isolation Dependability -
High-radiation isolation signal

II. One item submitted as confirmatory for Open Item 36 (open
item 36 is now closed)

1. Details of debris screen for 6" vent drywell line

II.B.3 - Procedures for the Determination of the Extent of Core Damage

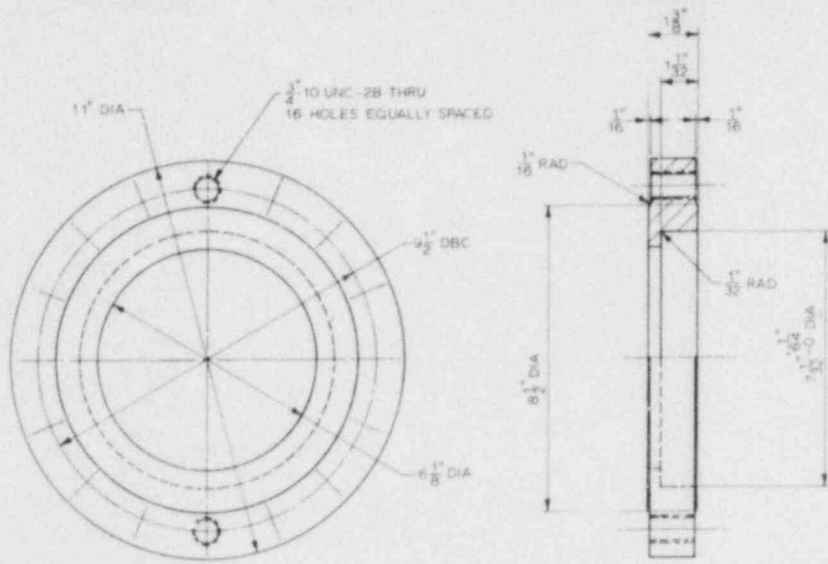
The attached document, "Procedures for the Determination of the Extent of Core Damage under Accident Conditions", RPE 81CCL01, has been prepared for possible use by members of the BWR Owners Group (BWROG). LILCO feels that utilization of the contents of this report is adequate to address the concept of determining the approximate degree of core damage based on measured fission product concentrations. The information obtained in utilizing these procedures is not considered essential as a determinant for potential operator action, although it could be used as guidance. It should be noted that the BWROG Regulatory Guide 1.97 Committee is presently addressing this concern. LILCO plans to adopt their resolution of this matter as appropriate after it is finalized but not necessarily prior to fuel load.

Item II.E.4.2 - Containment Isolation Dependability

LILCO commits to providing a high-radiation isolation signal to the 4" and 6" containment vent and purge isolation valves which may be open during operational conditions 1, 2 or 3. These valves are designated as follows:

1T24*AOV001A,B
1T24*AOV004A,B
1T46*AOV078A,B
1T46*AOV079A,B

This change will be implemented on a schedule consistent with equipment availability, but is not expected to be completed before commercial operation.



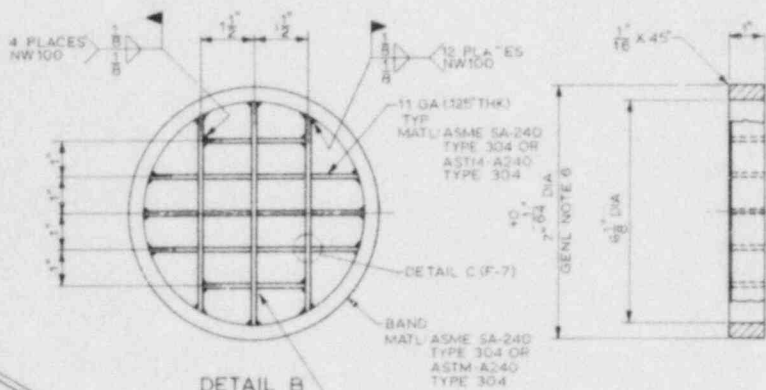
DET A
(B-B)
SCREEN SUPPORT FLG
HALF SIZE
MATL: ASME SA516 GR 70

GENERAL NOTES:

1. SCALE: AS NOTED
2. REMOVE ALL BURRS AND BREAK SHARP EDGES
3. ITEMS SHALL BE FABRICATED PER S & W SPEC SH1-056
4. FABRICATION AND WELDING BY FIELD FORCES
5. TORQUE SCREWS BY USING THE "TURN OF THE NUT METHOD" AS SPECIFIED IN THE AISC MANUAL OF STEEL CONSTRUCTION
6. MACHINE OD AFTER WELDING
7. DEBRIS SCREEN ASSY TO BE IDENTIFIED BY MARK NO. 1146#5-001

REFERENCE DRAWINGS:

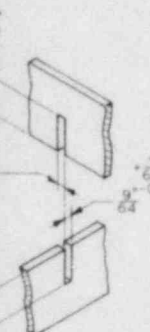
FLOW DIAGRAM SYS 1141
REACTOR BLDG-NORMAL VENT SYS M-10599 (FB-23A)



DETAIL B
(D-E)
SCREEN
HALF SIZE

DIM TOLERANCES UNLESS OTHERWISE SPECIFIED				
0 TO 6"	6" TO 1'-0"	1'-0" TO 2'-0"	2'-0" TO 3'-0"	ANGLES
MACHINING DIMENSIONS				
± 0.015	± 0.020	± 0.030	± 0.040	$\pm 0^{\circ}-30'$
GEN'L & WELDMENT DIMENSIONS				
± 0.030	± 0.040	± 0.060	± 0.125	$\pm 1^{\circ}-0'$

NUCLEAR SAFETY RELATED
QA CAT. I



DETAIL C
(H-I)
SLOT ARRGT
EXPLODED VIEW
11 PLACES
NTS

ENLARGED VIEW
11 PLACES
FULL SIZE

THIS DRAWING IS BASED ON
UNAPPROVED INPUT DOCUMENTS
11600.02-NW1(B)-290-N2C

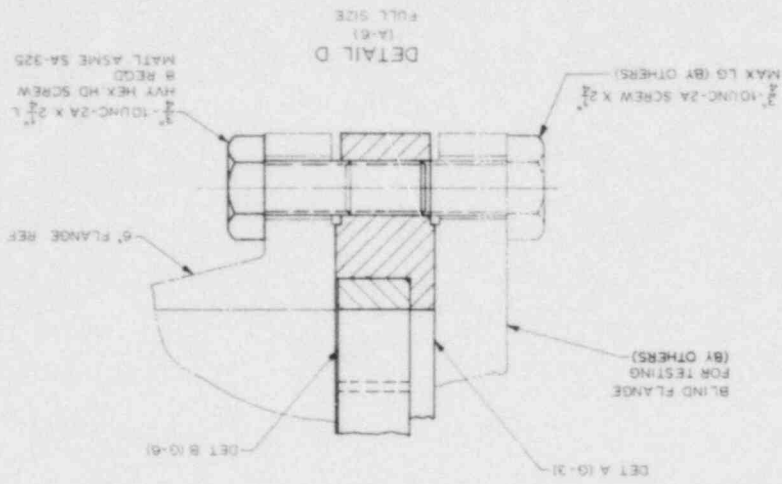
THE INFORMATION ON THIS DRAWING MAY NOT BE COPIED OR
USED FOR OTHER THAN THE CONSTRUCTION, MAINTENANCE OR
REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

APPROVED: *[Signature]*
LICENSED PROFESSIONAL ENGINEER NO. 17711 S
STATE OF NEW YORK

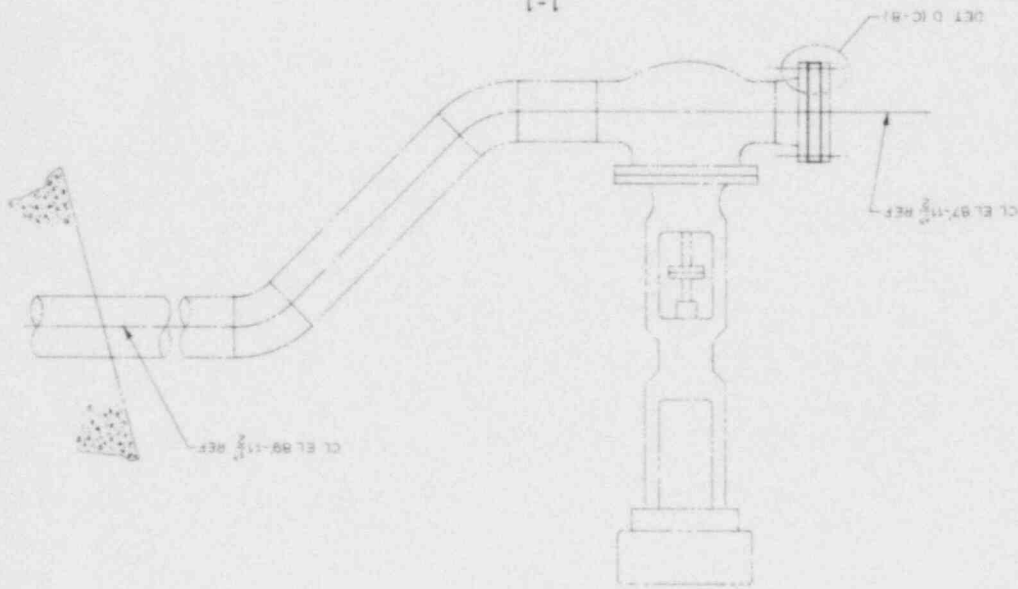
850 Mwe W.O. 48923

6" VENT LINE
DEBRIS SCREEN
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.
FILED
DRAWING NUMBER M-13607-1

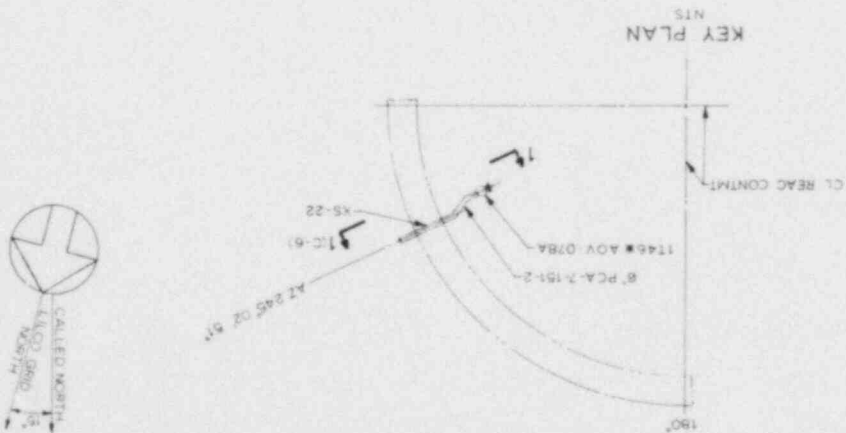
NO.	DESCRIPTION	QTY	UNIT	DATE	BY	CHKD	APPV	DATE	NO.	DESCRIPTION	QTY	UNIT	DATE	BY	CHKD	APPV	DATE	NO.	
1																			



SCALE: 1/2" = 1'-0"
 (2)
 1-1



KEY PLAN
 NTS



M-13607-1