

NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

December 14, 1981



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Docket No. 50-409 LS05-81-12-036

> Mr. Frank Linder General Manager Dairyland Power Cooperative 2615 East Avenue South La Crosse, Wisconsin 54601

Dear Mr. Linder:

SUBJECT: RESOLUTION OF NUREG-0737 ITEM II.K.3.14, ISOLATION CONDENSER

ISOLATION MODIFICATION

RE:

LA CROSSE BOILING WATER REACTOR

We have completed our review and evaluation of your response dated

September 3, 1980 regarding NUREG-0737 Item II.K.3.14, Isolation Condenser

Isolation Modification. The results of our review are discussed in the enclosed Safety Evaluation. Based on those results, we conclude that your response to the item is acceptable and that the item is considered resolved.

Sincerely,

Original signed by Thomas V. Wambach for/

Dennis M. Crutchfield, Chief Operating Reactors Branch #5 Division of Licensing

Enclosure: Safety Evaluation

cc w/enclosure: See next page

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Dennis M. Crutchfield, Chief Operating Reactors Branch #5 Division of Licensia

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SAFETY EVALUATION REPORT

APPLICATION OF NUREG-0737 ITEM II.K.3.14 - "ISOLATION OF ISOLATION CONDENSERS" TO OPERATING BWRS

INTRODUCTION

The requirements for automatic isolation of the isolation condensers on boiling water reactors was specified as a post-TMI requirement in NUREG-0737, which was transmitted to all licensees and applicants by letter dated October 31, 1980.

The position as stated in NUREG-0737 is:

"Isolation condensers have radiation monitors on their vents. These monitors provide alarms in the control room but do not isolate the isolation condensers. The isolation condensers are currently isolated on a high-radiation signal in the steam line leading to the isolation condensers. The design should be modified such that the isolation condensers are automatically isolated upon receipt of a high radiation signal at the vent rather than at the steam line. The purpose of the change is to increase the availability of the isolation condensers as heat sinks."

The six licensees for seven operating BWR units have responded to this item in References 1 - 3. This Safety Evaluation Report addresses the general need for the requirement for the seven operating units.

REGULATORY STAFF EVALUATION

The seven units covered by this review are Big Rock Point, Dresden 2/3, Millstone 1, Nine Mile Point, Oyster Creek and LaCrosse. All seven units have radiation monitors and alarms at the isolation condenser vents. Only the system design of Nine Mile Point Unit 1 incorporates the automatic isolation feature on high radiation at the vents. Nine Mile Point Unit 1 thus meets the requirements as stated and requires no further action. The remaining units require manual isolation of the 10 if considered necessary by the operator, or in some cases utilize other signals as indicators to the operator that some action is required. In addition, some of the units do not have isolation on high radiation in the steam

lines as considered in the NUREG-0737 staff position.

In general, the licensees responses (Reference 1-6) have taken the position that the design modification is not necessary, either because the unit design is atypical or that more flexibility and system availability is attained by manual isolation.

Based on our review of individual licensee submittals, the staff concludes that since the subject plants do not have isolation condenser isolation on high radiation signals in the steam lines, the design modification as specified in NUREG-0737 Item II.K.3.14 is not relevent and does not increase the availability of the ICs as heat sinks. The staff also agrees with the position of one licensee that manual isolation allows the operator a greater amount of flexibility and system availability to cope with all anticipated and unanticipated operation transients.

REGULATORY POSITION

We have reviewed the responses by six utilities to the NUREG-0737

Item II.K.3.14 requirements for automatic isolation of the isolation condensers on a high radiation signal at the IC atmospheric vents.

Based on the results of our review as discussed in the above staff evaluation, we conclude that the manual trip on high radiation levels at the vents is sufficient to provide the amount of flexibility and system availability intended by the NUREG-0737 requirement.

We conclude that the licensees' present positions, as stated in their respective submittals, are acceptable.