Mailing Address

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F. L. Clayton, Jr. Senior Vice President Flintridge Building



December 22, 1981

Docket No. 50-348 50-364

Director, Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Mr. S. A. Varga

Gentlemen:

Joseph M. Farley Nuclear Plant - Units 1 and 2 NRC Commitment Compliance Status

In order to provide clarification status and required submittals as related to various Post-TMI issues; the Unit 2 license; and other NRC requirements, Alabama Power Company provides the following enclosures.

Enclosures 1 and 3 provide the status of NUREG-0737 requirements for Units 1 and 2, respectively. Enclosures 2 and 4 provide the status of the NUREG-0737 Technical Specification requirements for Units 1 and 2, respectively. Enclosure 5 provides the status of the Unit 2 license requirements. Enclosure 6 provides the status of other NRC requirements for January 1, 1982; the 3rd Refueling Outage for Unit 1; the 1st Refueling Outage for Unit 2. Enclosure 7 includes the basis for each extension indicated in Enclosures 1 through 6 and provides all submittals not previously provided.

If you have any questions, please advise.

Yours very truly,

Clayton, Jr.

FLCJr/JAR:jc Enclosures cc: Mr. R. A. Thomas (w/enclosures) Mr. G. F. Trowbridge (w/enclosures) Mr. J. P. O'Reilly (w/enclosures) Mr. E. A. Reeves (w/enclosures) Mr. W. H. Bradford (w/enclosures)

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ENCLOSURE 1

J. M. FARLEY NUCLEAR PLANT UNIT 1 STATUS OF NUREG-0737 REQUIREMENTS

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|--|---|---------------------------------|------------------------|--------------------------------------|------------------------------|-----------------|------------------------|
| I.A.1.1 | Shift technical advisor | 2. Tech specs | 12-15-80 | Yes | | 12-15-80 | Complete | N/A |
| | | 3. Training per LL Cat B | 1-1-81 | Yes | | 1-14-81 2-5-81 | Complete | N/A |
| | | Describe long- term program | 1-1-81 | No | | 1-14-81 2-5-81 | Complete | N/A |
| I.A.1.3 | Shift manning | 1. Limit overtime | 11-1-80 | No | | 6-26-80 | Complete | N/A |
| | | 2. Minimum shift crew | 7-1-82 | Yes | Amend TS on shift manning | 6-26-80 | Complete | N/A |
| I.A.2.1 | Immediate upgrade of RO & SRO training and | 1. SRO experience | 5-1-80 | No | Completion to be verified | 7-10-80 | Complete | N/A |
| | | 2. SROs be ROs, 1 yr. | 12-1-80 | No | Completion by OIE | 1-14-81 | Complete | N/A |
| | | 3. 3 mo training on-shift | 8-1-80 | No | Completion by OIE | 1-14-81 | Complete | N/A |
| | | 4. Modify training | 8-1-80 | No | NRR staff to review | 7-10-80 1-14-81 | Complete | N/A |
| | | 5. Facility certi- fication | 5-1-80 | No | OIE veri- fication | 7-10-80 | Complete | N/A |
| I.A.2.3 | Administration of training programs | Instructors com- plete SRO exam | 8-1-80 | No | NRR to verify confor- mance | 1-14-81 | Complete | N/A |

N/A - There are no further requirements for this item.
No - Remaining requirements are expected to be completed on schedule
Yes - An extension for this item was requested in the letter listed below.

Encl. 1

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|--|---|------------------------|----------------------------------|---|--|------------------------|
| I.A.3.1 | Revise scope and criteria for Ticensing exams | 1. Increase scope | 5-1-80 | No | | 7-10-80 | Complete | N/A |
| | | 2. Increase pass- ing grade | 5-1-80 | No | | 7-10-80 | Complete | N/A |
| | | 3. Simulator exams b. All | 10-1-81 | No | Plants w/o simu- lators | 1-14-81 2-9-81 | Complete (Plant specific simulator exams sche- duled to be given by 7-1-83) | N/A |
| 1.C.1 | Short-term accident and procedure review | Inadequate core cooling a. Reanalyze and propose guidelines | 1-1-81 | No | | 1-14-81 W.O.G letter 80-179, dated 12-15-80 | Complete | N/A |
| | | b. Revise procedures | First refueling outage afte 1/1/82 | No er | | 1-14-21 9-9-81 (not applicable due to early outage) | Suspense: 4th Refuel Outage See Encl. 7 item 1 | No |
| | | Transients & accidents a. Reanalyze & propose guidelines | 1-1-81 | No | | 1-14-81 | Complete | N/A |
| | | b. Revise procedures | First refueling outage afte 1/1/82 | No | | 1-14-81 | Suspense: 4th Refuel Outage See Encl. 7 | No |

Encl. 1 2

Encl. 1 3

UNIT 1 NUREG-0737 STATUS

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|---|---------------------------------|------------------------|---|------------------------------|--|------------------------|
| I.C.5 | Feedback of operating experience | Licensee to imple- ment procedures | 1-1-81 | No | | 1-14-81 | Complete | N/A |
| 1.C.6 | Verify correct performance of operating activities | Revise performance procedures | 1-1-81 | No | | 2-5-81 2-23-81 | Complete | N/A |
| I.D.1 | Control-room design reviews | Preliminary assessment & schedule for correcting deficiencies | TBD | | Final guidance will be issued 1981 as NUREG-0700 | 2-23-81 | Awaiting finaliza- tion of require- ments by NRC See Encl. 7 item 2 | No 7, |
| I.D.2 | Plant-safety- parameter display console | 1. Description | TBD | | Guidance per NUREG- 0696 Rev. 2 | 1-14-81 11-16-81 | Awaiting finaliza- tion of require- ments by NRC | No |
| | | 2. Installed | TBD | | Guidance per NUREG- 0696 Rev. 2 | 1-14-81 11-16-81 | Awaiting finaliza- tion of require- ments by NRC | No |
| | | 3. Fully imple- mented | TBD | - | Guidance per NUREG- 0696 Rev. 2 | 1-14-81 11-16-81 | Awaiting finaliza- tion of require- ments by NRC | No |

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | .ech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|----------------------------------|--|---------------------------------|------------------------|----------------|------------------------------|--|------------------------|
| II.B.1 | Reactor-coolant- system vents | <pre>2. Install vents (LL Cat B)</pre> | 7-1-82 | Yes | | 1-14-81 | Complete | N/A |
| | | 3. Procedures | 1-1-82 | Yes | | 1-14-81 | Suspense: when vent system is operational and NRC ap- proved See Encl. 7 item 3 | No 1 7, |
| II.B.2 | Plant shielding | <pre>2. Plant modi- fications (LL Cat B)</pre> | 1-1-82 | No | | 2-23-81 | Complete except for the elec- trical dis- connect devices which are scheduled to be in- stalled prior to startup following the current refueling outage See Encl. 7 item 14 | N/A |
| | | Equipment qualification | 6-30-82 | No | | 2-23-81 9-9-81 | Suspense: 6-30-82 | No |
| II.B.3 | Postaccident sampling | Plant modifi- cations (LL Cat B) | 1-1-82 | Yes | | 1-14-81 | Complete | N/A |

Encl. 1

Encl. 1

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UNIT 1 NUREG-0737 STATUS

| Clarifi- cation Item | Shortened | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|---|---------------------------------|------------------------|----------------|------------------------------|---|------------------------|
| II.B.4 | Training for mitigating core damage | 1. Develop train- ing program | 1-1-81 | No | | 2-9-81 | Complete | N/A |
| | | 2. Implement | | | | | | |
| | | program a. Initial | 4-1-81 | No | | 2-9-81 | Complete | N/A |
| | | b. Complete | 10-1-81 | No | | 2-9-81 | Complete | N/A |
| II.D.1 | Relief and | 2. RV & SV testing | | | | | | |
| | test requirements | a. Complete testing | 7-1-81 | No | | 6-25-81 9-30-81 | Additional information to be pro- vided upon completion of the EPRI program | No n |
| | | b. Plant speci- fic report | 10-1-81 | TBD | | 6-25-81 9-30-81 | Additional information to be pro- vided upon completion of the EPRI program | No 1 |
| | | Block valve testing | 7-1-82 | TBD | | 2-23-81 | Suspense: 7-1-82 APCo is participa- ting in EPRI sub- mittal | No |
| II.D.3 | Valve position | 2. Tech Spec | 12-15-80 | Yes | | 12-15-81 | Complete | N/A |



| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. NRC Req. Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|--|---------------------------------|------------------------------------|------------------------------|--|------------------------|
| II.E.1.1 | Auxiliary feed- water system evaluation | 1. Short term | 7-1-81 | Item specific | 1-14-81 | Complete | N/A |
| | | 2. Long term | 1-1-82 | Item specific | 1-14-81 | Complete | N/A |
| II.E.1.2 | Auxiliary feedwater system initiation and flow | 1. Initiation (b) Safety grade | 7-1-81 | Yes | 2-9-81 7-1-81 | Suspense: 4th Refuel Outage See Encl 7, item 4 | Yes 7-1-81 |
| | | Flow indication (b) LL Cat A tech specs | 12-15-80 | Yes | 12-15-80 | Complete | N/A |
| | | (c) Safety grade | 7-1-81 | Yes | 2-9-81 7-1-81 | Suspense: 4th Refuel Outage See Encl 7, item 4 | Yes 7-1-81 |
| II.E.3.1 | Emergency power for pressurizer heaters | 2. Tech specs | 12-15-80 | Yes II.G.1 | 12-15-80 | Complete | N/A |
| II.E.4.1 | Dedicated hydro- gen penetrations | 2. Install | 7-1-81 | No | 1-14-81 | Not required | N/A |
| II.E.4.2 | Containment isolation dependability | 5. Containment press setpoint | | | 2-13-81 9-9-81 | Complete | N/A |
| | sependability | a. Specify pressure | 1-1-31 | No | | | |

b. Modifica-tions 7-1-81 ies

Encl. 1

| Clar fi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. NRC Req. Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|--|--|---------------------------------|------------------------------------|------------------------------|----------------------------------|------------------------------------|
| | | Containment purge valves | 1-1-81 | Yes | 1-14-81 | Complete | N/A |
| | | Radiation signal on purge valves | 7-1-81 | Yes | 1-14-81 | Complete | N/A |
| | | 8. Tech Specs | 12-15-80 | Yes | 12-15-80 | Complete | N/A |
| II.F.1 | Accident- monitoring | 1. Noble gas monitor | 1-1-82 | Yes | 2-13-81 | Complete See Encl 7 item 5 | » //А |
| | | Iodine/ particulate sampling | 1-1-82 | Yes | 2-13-81 | Complete See Encl 7 item 5 | N/A |
| | | Containment high range monitor | 1-1-82 | Yes | 2-13-81 | Complete | N/A |
| | | Containment pressure | 1-1-82 | Yes | 2-13-81 | Complete | See Encl 2, item II.F.1.4 |
| | | 5. Containment water level | 1-1-82 | Yes | 2-13-81 | Complete | N/A |
| | | Containment hydrogen | 1-1-82 | Yes | 2-13-81 | Complete | N/A |
| II.F.2 | Instrumentation for detection of inadequate core- cooling | 2. Tech Spec (LL Cat A) | 12-15-80 | Yes | 12-15-80 | Complete | See Encl 2, item II.F.2 |

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|--|--|---------------------------------|------------------------|--|---|---|------------------------|
| | | <pre>3. Install level instruments (LL Cat B)</pre> | 1-1-82 | Yes | | 3-14-80 2-9-81 6-24-81 6-29-81 10-6-81 12-31-79 1-14-81 | Complete (Incore the mocouple panel test to be per- formed prior to startup following current refuel outage) | N/A er- |
| II.G.1 | Power supplies for pressurizer relief valves, block valves, & level indicators | 2. Tech specs | 12-15-80 | Yes | See II.E.3.1 | 12-15-80 | Complete | N/A |
| 11.K.1 | IE Bulletins | 79-05,06,08 | Bulletin Specific | No | NRR eva- luating licensee responses | 6-22-79 | Complete | N/A |
| II.K.2 | Orders on B&W Plants | 13. Thermal- mechanical report | 1-1-82 | As require | ed | 1-14-81 W.O.G. 81-138 dated 4-1-81 5-20-81 | See Encl 7, item 6 | . No |
| | | 17. Voiding in RCS | 1-1-82 | No | | 1-14-81 | Complete See Encl 7, item 7 | N/A |
| | | 19. Benchmark analysis seq AFW flow | 1-1-82 | No | | 1-14-81 NRC letter 7-6-81 | Not required | N/A |

Encl. 1 8

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|--------------------------------------|---|---|------------------------|--|---------------------------------|--|------------------------|
| II.K.3 | Final recommenda- tions, B&O task | 1. Auto PORV isolation | | | | 5-26-81 | Not required | N/A |
| | force | a. Design | 7-1-81 | Yes | <pre>if required by II.K.3.2</pre> | | | |
| | | b. Test/install | lst refuel 6 mo. after staff appro val | Yes - | | | | |
| | | 2. Report on PORV failures | 1-1-81 | No | | 1-14-81 WCAP-9804 3-13-81 | Complete | N/A |
| | | Reporting SV & RV failures & challenges | 1-1-81 | Yes | Initiate data begin- ning 4-1-80 | 1-14-81 9-9-81 | System in place | N/A |
| | | 5. Auto trip of RCPs | | | | | | |
| | | a. Propose mo- difications | 7-1-81 | No | | 6-30-81 | Suspense: Within 3 months after NRC determina- tion of accepta- bility of SB LOCA Model See Encl. 7 item 15 | No |

Encl. 1

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo E Remarks R | xtension equested |
|----------------------------|--------------------|---|---------------------------------|------------------------|--|------------------------------|---|----------------------|
| | | b. Modify | 3-1-82 | Yes | If required | 6-30-81 | Suspense: Within 11 months after NRC determina- tion of accepta- bility of SB LOCA model during an appro- priate out- age or 1st such outage thereafter | СИ |
| | | 9. PID controlle | r 1-1-81 | No | Implemen- tation to be verified | 6-26-80 | Complete | N/A |
| | | Proposed anticipatory trip modifi- cations | Plant specific | Yes | | 6-26-80 | Complete | N/A |
| | | <pre>11. Justify use of certain PORVs</pre> | Plant specific | No | See NUREG- 0611, Sect. 3.2.4.d. | 1-14-81 | Complete | N/A |
| | | 12. Anticipatory trip on tur- bine trip a. Confirma- tion or propose modifica- tions | 1-1-81 | No | | 6-26-80 | Complete | N/A |

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. NRC Req. Remarks | APCo Response Letter Date | APCo E) Remarks Re | tension equested |
|----------------------------|--------------------|---|---|------------------------------------|---|--|---------------------|
| | | b. Modify | lst refuel or 6 mo. after staff approval | Yes | 6-26-80 | Complete | N/A |
| | | 17. ECCS outages | 1-1-81 | As required | 2-13-81 | Suspense: Every 5 year period after initial criticality | No |
| | | 25. Power on pump | | | | | |
| | | a. Propose mods | 1-1-82 prior to SER | No | 1-14-81 | Complete | N/A |
| | | b. Modifica- tions | 7-1-82 | No | 1-14-81 | Complete See Encl 7, item 8 | N/A |
| | | 30. SB LOCA methods a. Schedule outline | 11-15-80 | No | Westinghouse letter dated 11-25-81 NS-EPR-2524 | Complete | N/A |
| | | b. Model | 1-1-82 | No | Westinghouse letter dated 11-25-81 NS-EPR-2524 | Complete See Encl 7, item 9 | N/A |
| | | c. New analyses | 1-1-83 or 1 yr after staff appro val. | No)- | Westinghouse letter dated 11-25-81 NS-EPR-2524 | Complete See Encl 7, item 9 | N/A |

Encl. 1

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|---|---|------------------------|-----------------------------------|------------------------------|----------------------|------------------------|
| | | 31. Compliance with 10 CFR 50.46 | 1-1-83 or 1 yr after staff appro val | TBD | | 1-14-81 | Suspense: 1-1-83 | No |
| III.A.1.2 | Upgrade emer- gency support facilities | 2. Design | TBD | TBD | | 2-13-81 5-19-81 | Complete | N/A |
| | | 3. Modifications | TBD | TBD | | 2-13-81 5-19-81 | Suspense: 10-1-82 | No |
| III.A.2 | Emergency preparedness | Upgrade emer- gency plans to App E, 10 CFR 50 | 3-1-81 | Yes | Procedures submitted 3-1-81 | 11-7-80 | Complete | N/A |
| | | Meteorological data | 6-1-83 | Yes | Staged implemen- tation | 2-9-81 | Suspense: 7-1-82 | No |
| III. 0. 1 | Primary coolant outside con- tainment | 2. Tech specs | 12-15-80 | Yes | | 12-15-80 | Complete | N/A |
| III.D.3.3 | Inplant I ₂ radiation monitoring | Modifications to accurately measure I₂ | 1-1-81 | Yes | | 1-14-81 9-9-81 | Complete | N/A |
| III.D.3.4 | Control-room habitability | 1. Review | 1-1-81 | No | | 6-26-80 | Complete | N/A |
| | | 2. Modifications | TBD | Yes | | 6-26-80 | Not required | N/A |

Encl. 1

ENCLOSURE 2

J. M. FARLEY NUCLEAR PLANT UNIT 1

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

J. M. FARLEY NUCLEAR PLANT UNIT 1

4

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED |
|--------------|---------------------------------|---|--|
| I.A.1.1 | Shift Technical Advisor | Presently in FNP-1 Tech Spec. Section 6.2.3 and Table 6.2.1. Also, part of FNP-1 Tech. Spec. Up- grade Package Section 6.2.4, Figure 6.2-2 and Table 6.2-1. | |
| I.A.1.3 | Shift Manning | Presently in FNP-1 Tech. Spec. Section 6.2.2. and Table 6.2.1. Also, part of FNP-1 Tech. Spec. Up- grade Package Section 6.2.2 and Table 6.2-1. | |
| 11.8.1 | Reactor Coolant System Vents | Pressurizer Vent (PORV) is in FNP-1 Tech. Spec. 3.4.4.a. Reactor Vessel Head Vents are not in FNP-1 Tech. Spec. Also, Pressurizer Vent (PORV) is part of FNP-1 Tech. Spec. Upgrade Package Section 4.5. | Reactor Vessel Head Vents are installed A Tech. Spec. will be written upon NRC approval of installed system. |
| II.B.3 | Postaccident Sampling | Not currently in FNP-1 Tech. Spec. Part of FNP-1 Tech. Spec. Upgrade Package Section 6.8.3.d. | |
| II.D.3 | Valve Position Indication | Presently in FNP-1 Tech. Spec. Tables 3.3-11 and 4.3.7. Also, part of FNP-1 Tech. Spec. Up- grade Package Tables 3.3.11 and 4.3.7. | |

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 <u>ITEM</u> | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION PEQUESTED |
|---------------------|--|---|-----------------------------|
| II.E.1.2 | Анхilliary Feed- Water System Initiation & Flow | Initiation is part of current FNP-1 Tech. Spec. Tables 3.3-3, 3.3-4, 3.3-5 and 4.3-2. Flow is part of current FNP-1 Tech. Spec. Tables 3.3-11 and 4.3-7. These Tech. Specs. are also part of the FNP-1 Tech. Spec. Upgrade Package. | |
| II.E.3.1 | Emergency Power for Pressurizer Heaters | Presently in FNP-1 Tech. Spec. Section 3.4.4. Also, part of FNP-1 Tech. Spec. U. grade Package Section 3/4.4. | |
| II.E.4.2 | Containment Isolation Dependability | Containment Isolation Dependability is part of current FNP-1 Tech. Spec. Section 3.3.2.1, 3.3.3.1 and 3.6.3. These same Technical Specifications are also part of the FNP-1 Tech. Spec. Upgrade Package. Containment purge valves are part of FNP-1 Tech. Spec. Upgrade Package 3.6.1.7 | |
| II.F.1 | Accident- Monitoring | Noble Gas Monitor is part of FNP-1 Tech. Spec. Upgrade Package Table 3.3-6. Not currently in FNP-1 Tech. Spec. | |
| | | Iodine/particulate sampling is contained in FNP-1 Tech. Spec. Section 6.8.3.b and is also part of FNP-1 Tech. Spec. Upgrade Package Section 6.8.3.d. | |

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J. M. FARLEY NUCLEAR PLANT UNIT 1

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 1TEM | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED |
|-----------------------|---|---|---|
| II.F.1 (continued) | | Containment High-Range Monitor is part of FNP-1 Tech. Spec. Upgrade Package Table 3.3-6. Not currently in FNP-1 Tech. Spec. | |
| | | Containment Pressure is not in FNP-1 Tech. Spec. | Containment pressure monitor has been installed and is operational and is included in the current Unit 1 Upgrade to the Tech. Spec. |
| | | Containment water level is part of FNP-1 Tech. Spec. Upgrade Package Table 3.3-11 and Table 4.3-7. Not currently in FNP-1 Tech. Spec. | |
| | | Containment Hydrogen is part of current FNP-1 Tech. Spec. Section 3.6.4. | |
| 11.F.2 | Instrumentation for Detection of Inadequate Core Cooling | Subcooling meter is presently in FNP-1 Tech. Spec. Table 3.3-11 and 4.3-7. Subcooling meter and incore thermocouples are part of FNP-1 Tech. Spec. Upgrade Package Table 3.3-11 and 4.3-7. Reactor vessel water level is not in FNP-1 Tech. Specs. | Reactor vessel water level is to be in- corporated in Tech. Specs. upon approval by NRC of the FNP "Special" level system. However, the BF3 prototype system is currently under a testing pro- gram described in APCo's letter of 6-29-81. Upon completion of this test |
| II.G.1 | Power Supplies for Pressurizer Relief Valves, | Presently pressurizer level indication and block valves are part of current FNP-1 Tech. Spec. Sections 3.8.2.1 and 3.8.2.2, and PORVs in Sections | will be developed. |

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED | |
|-----------------------|--|---|-----------------------------|--|
| II.G.1 (continued) | Block Valves, & Indicators | 3.8.2.3 and 3.8.2.4. These Tech. Specs. are also part of the FNP-1 Tech. Spec. Upgrade Package. | | |
| II.K.3 | Final Recommendations B & O Task Force | Reporting SV & RV Failures & Challenges is part of FNP-1 Tech. Spec. Upgrade Package Section 6.9.1.10. Not in current FPN-1 Tech. Spec. | | |
| | | 10 & 12. Anticipatory Trip on turbine trip is part of current FNP-1 Tech Spec. Table 3.3-1. These Tech. Specs are also part of the FNP-1 Tech. Spec. Upgrade Package. | | |
| III.A.2 | Emergency Preparedness | Presently is Part of current FNP-1 Tech. Spec. Sect 6.8.1. These Tech. Spec. are also part of the FNP-1 Tech. Spec. Upgrade Package. | Ion | |
| 111.D.1.1 | Primary Coolant Outside Contain- ment | Presently is part of current FNP-1 Tech. Spec. Sect 6.8.3.a. These Tech. Specs. are a so part of the FNP-1 Tech. Spec. Upgrade Package. | ion | |
| 111.D.3.3 | Inplant Radiation Monitoring | Presently in part of current FNP-1 Tech. Spec. Upgra Package Section 6.8.3.b. These Tech. Specs. are also part of the FNP-1 Tech. Spec. Upgrade Package. | ade | |

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 | SHORTENED |
|------|-----------|
| ITEM | TITLE |

SPECIFICATION SECTION

COMMENT/EXTENSION REQUESTED

III.D.3.4 Control-Room Habitability

1.2.1.1

Presently is part of current FNP-1 Tech. Spec. Section 3.3.3.6 and 3.7.7.1. These Tech. Specs. are also part of the FNP-1 Tech. Spec. Upgrade Package. ENCLOSURE 3

J. M. FARLEY NUCLEAR PLANT UNIT 2

STATUS OF

NUREG-0737 REQUIREMENTS

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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | AFCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|--|---|---------------------------------|------------------------|----------------|------------------------------|-----------------|------------------------|
| I.A.1.1 | Shift technical advisor | 1. On shift | Fuel Load | Yes | | 1-14-81 | Complete | N/A |
| | | 2. Training per LL Cat B | Fuel Load | No | | 1-14-81 | Complete | N/A |
| | | Describe long- term program | Fuel Load | No | | 1-14-81 | Complete | N/A |
| I.A.1.2 | Shift supervisor responsibilities | Delegate nonsafety duties | Fuel Load | No | | 6-20-80 | Complete | N/A |
| I.A.1.3 | Shift manning | 1. Limit overtime | Fuel Load | No | | 2-23-81 | Complete | N/A |
| | | 2. Minimum shift crew | Fuel Load | Yes | Case by case | 1-14-81 | Complete | 97A |
| I.A.2.1 | Immediate upgrade of RO & SRO training and qualifications | 1. SRO experience | Fuel Load | No | | 1-14-81 | Complete | N/A |
| | | 2. SROs be ROs, 1 yr. | Initial Criticality | No | | 1-14-81 | Complete | N/A |
| | | 3 mo training on-shift | Fuel Load | No | | 1-14-81 | Complete | N/A |
| | | 4. Modify training | Fuel Load | No | | 1-14-81 | Complete | N/A |
| | | 5. Facility certi- fication | Fuel Load | No | | 1-14-81 (I.A.2.3) | Complete | N/A |

N/A - There are no further requirements for this item.
No - Remaining requirements are expected to be completed as scheduled.
Yes - An extension for this item was requested in the letter listed below.

Encl. 3

UNIT 2 NUREG-0737 STATUS Encl. 3 2 Clarifi-Imp'emen-Tech. cation Shortened tation Spec. NRC APCo Response APCO Extension Title Description Remarks Letter Date Remarks Schedule Req. Requested Item 2 mo prior Complete N/A I.A.2.3 Administration Instructors com-NO 1-14-81 of training plete SRO exam to issuance of license programs I.A.3.1 Revise scope and 1. Increase scope 8-1-80 Complete 10-1-80 No N/A criteria for licensing exams 2. Increase pass-10-1-80 8-1-80 Complete N/A No ing grade 3. Simulator exams 10-1-81 N/A No 1-14-81 Complete b. All plants 2-9-81 (Plant specific simulator exams to be given by 7-1-83) 8-8-80 I.B.1.2 Evaluation of Organization, Fuel Load Yes Draft Complete N/A guideline 1-14-81 organization resources tng. & and management qualifications available for operators & accidents I.C.1 1. SB LOCA Fuel Load No 10-24-79 Complete N/A Short-term 1-14-81 accident and procedure review 10-24-79 N/A 2. Inadequate Fuel Load No Complete core cooling 1-14-81 a. Reanalyze and propose guidelines

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo E Remarks F | 3 Extension Requested |
|----------------------------|--|--|--|------------------------|----------------|------------------------------|---|-----------------------------|
| | | b. Revise procedures | First refueling outage after 1/1/82 | No | | 1-14-81 | Suspense: end of 1st refuel outage See Encl. 7 item 1 | No |
| | | Transients & accidents a. Reanalyze & propose guidelines | Fuel Load | No | | 10-24-79 1-14-81 | Complete | N/A |
| | | b. Revise procedures | First refueling outage after 1/1/82 | No | | 1-14-81 | Suspense end of 1st refuel out- age See Encl 7, item 1 | No |
| 1.C.2 | Shift & relief turn- over procedures | Revise procedures to assure plant status known by new shift | Fuel Load | No | | 6-20-80 | Complete | N/A |
| I.C.3 | Shift supervisor responsibility | Corporate direc- tive to establish command duties and revise plant procedures | Fuel Load | Yes | | 6-20-80 | Complete | N/A |
| 1.C.4 | Control-room access | Establish author- ity and limit access | Fuel Load | No | | 6-20-80 | Complete | N/A |
| I.C.5 | Feedback of operating experience | Review & revise procedures | Prior to issuance of OL | No | | 1-14-81 | Complete | N/A |

Encl. 3

19

| Clarifi- | | | Implemen- | Tech | | | | 4 |
|----------------|---|---|-------------------------------|---------------|---|------------------------------|--|--------------------------------|
| cation Item | Shortened Title | Description | tation Schedule | Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
| 1.C.6 | Verify correct performance of operating activities | Revise performance procedures | Fuel Load | No | | 2-23-81 | Complete | N/A |
| I.C.7 | NSSS vendor rev of proc | Low-power test program | Fuel Load | No | | 2-9-81 | Complete | N/A |
| | | Power ascension and emergency procedures | Full Power | No | | 2-9-81 | Complete | N/A |
| 1.C.8 | Pilot mon. of selected emer- gency proc for NTOLs | Correct procedure based on NRC sample audit | Full Power | No | | 6-30-80 | Complete | N/A |
| I.D.1 | Control-room design reviews | Preliminary assessment & schedule for correcting deficiencies | Prior to issuance of OL | No | Guidance and schedule being developed | 2-23-81 | Awaiting finalizat of requir ments by See Encl item 2 | No tion Pe- NRC 7, |
| I.D.2 | Plant-safety- parameter display console | 1. Description | TBD | No | Guidance and sche- dule being developed in NUREG- 0696 | 1-14-81 11-16-81 | Awaiting finaliza- tion of require- ments by NRC | No |
| | | 2. Installed | TBD | No | Guidance and sche- dule being developed in NUREG- 0696 | 1-14-81 11-16-81 | Awaiting finaliza- tion of require- ments by NRC | No |

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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | 5 Extension Requested |
| | | 3. Fully imple- mented | TBD | No | Guidance and sche- dule being developed in NUREG- 0696 | 1-14-81 11-10-81 | Awaiting finaliza- tion of require- ments by NRC | No |
| I.G.1 | Training during low-power testing | 1. Propose tests | Fuel Load | No | | 6-20-80 | Complete | N/A |
| | | Submit analysis and procedures | Fuel Load | Yes | | 9-2-80 9-11-80 | Complete | N/A |
| | | Training and results | Full Power | No | | 5-18-81 | Complete | N/A |
| II.B.1 | Reactor-coolant- system vents | 1. Design and analysis | Full Power | No | | 1-14-81 | Complete | N/A |
| | | 2. Install | 7-1-82 | Yes | | 1-14-81 | Complete | N/A |
| | | 3. Procedures | 1-1-82 | Yes | | 1-14-81 | Suspense: when vent system is operationa and NRC ap roved See Encl. item 3 | No 1 0p- 7, |
| II.B.2 | Plant shielding | Radiation and shielding review | Four months before 0.L. | No | | 1-14-81 | Complete | N/A |
| | | 2. Correcting | Full Power | No | | 1-14-81 | Complete | N/A |

actions to assure access

| | | | UNIT 2 NUREG- | -0737 STAT | US | | | Enc1. 3 |
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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | 6 Extension Requested |
| | | 3. Complete mods | 1-1-82 | No | | 2-23-81 | Suspense: 4-1-82 The Unit 2 Full Power License requires this item by 4-1-82. See Encl 7 item 14 | Yes |
| | | Equipment qualification | Four months before 0.L. | No | | 2-23-81 | Suspense: 4-1-82 | No |
| II.B.3 | Postaccident sampling | 1. Design review | Four months before 0.L. | No | | 2-9-81 | Complete | N/A |
| | | 2. Corrective actions | Full Power | Yes | | 2-9-81 | Complete | N/A |
| | | 3. Procedures | Full Power | Yes | | 2-9-81 | Complete | N/A |
| | | Complete actions | 1-1-82 | Yes | | 2-9-81 | Complete | N/A |
| II.8.4 | Training for mitigating core damage | 1. Develop train- ing program | Fuel Load | No | | 2-9-81 | Complete | N/A |

| | | | UNIT 2 NUREG- | -0737 STAT | rus | | | Encl. 3 |
|----------------------------|---|---|------------------------------------|-----------------------|----------------|------------------------------|--|-----------------------------|
| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | 7 Extension Requested |
| | | 2. Complete training | Full Power | No | | 2-9-81 | Complete | N/A |
| II.D.1 | Relief and safety-valve test requirements | 1. Describe pro- gram & schedule | Fuel Load | No | | 7-17-80 | Complete | N/A |
| | | 2. RV & SV tests | Fuel Load | TBD | | 2-9-81 6-25-81 9-30-81 | Additiona informati to be pro vided upo completio of the EP program. | 1 N/A on n RI |
| | | Block valve tests | 7-1-82 | TBD | | 2-9-81 | Suspense: 7-1-82 | No |
| II.D.3 | Valve position indication | Install in control room | Four months before 0.L. | Yes | | 1-14-81 | Complete | N/A |
| II.E.1.1 | Auxiliary feed- water system evaluation | 1. Analysis | Full Power | No | | 6-20-81 | Complete | N/A |
| | | 2. Modification | Full Power | No | | 1-14-81 | Complete | N/A |
| II.E.1.2 | Auxiliary feedwater system initiation and flow | Initiation (a) Control grade (b) Safety grade | Four months before Fuel Load | Yes | | 2-9-81 7-1-81 | Suspense: 1st Refue Outage | No 1 |
| | | <pre>2. Flow indication (a) Control grade (b) Safety grade</pre> | Four months before Fuel Load | Yes | | 2-9-81 7-1-81 | Suspense: 1st Refue Outage | No 1 |

| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------------|---|--|---|------------------------|----------------|------------------------------|--------------------------------|------------------------|
| II.E.3.1 | Emergency power for pressurizer heaters | Installed capa- bility | Four months prior to issuance of SER | Yes | | 1-14-81 | Complete | N/A |
| II.E.4.1 | Dedicated hydro- gen penetrations | 1. Design | Four months before 0.L. | Yes | | 1-14-81 | Not required | N/A |
| | | 2. Review and revise H ₂ control proc | Fuel Load | No | | 1-14-81 | Complete | N/A |
| | | 3. Install | 7-1-81 | No | | 1-14-81 | Complete | N/A |
| II.E.4.2 | Containment isolation dependability | 1-4 Implement diverse isolation | Prior to issuance of OL | Yes | | 1-14-81 | Complete | N/A |
| | | 5. Containment press setpoint | 7-1-81 | Yes | | 2-13-81 | Complete | N/A |
| | | Containment purge valves | Prior to issuance of OL | Yes | | 1-14-81 | Complete | N/A |
| | | Radiation signal on purge valves | 7-1-81 | Yes | | 1-14-81 | Complete | N/A |
| II.F.1 | Accident- monitoring instrumentation | 1. Procedures | Fuel Load | No | | 7-24-80 | Complete | N/A |
| | | 2. Install instru- mentation a. Noble gas monitor | 1-1-82 | Yes | | 2-13-81 | Complete See Encl item 5 | N/A 7, |
| | | b. Iodine/ particulate sampling | 1-1-82 | Yes | | 2-13-81 | Complete See Encl item 5 | N/A 7, |

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Encl. 3

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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response _Letter Date | APCo Remarks | Extension Requested |
| | | c. Containment high range monitor | t 1-1-82 | Yes | | 2-13-81 | Complete | N/A |
| | | d. Containment pressure | t ອິ mo prior to issuance of OL | Yes | | 2-13-81 | Complete | See Encl 4, item II.F.1.d |
| | | e. Containment water level | 7-1-82 | Yes | | 2-13-81 | Complete | N/A |
| | | f. Containment hydrogen | 1-1-82 | Yes | | 2-13-81 | Complete | N/A |
| II.F.2 | Instrumentation for decection of inadequate core- cooling | 1. Procedures instruments | Fuel Load | No | | 6-20-80 | Complete | N/A |
| | | 2. Subcooling meter | Fuel Load | Yes | | 6-20-80 | Complete | N/A |
| | | Describe other instrumentation | r Fuel Load | No | | 6-20-80 | Complete | N/A |
| | | Install addi- tional instru- menation | 1-1-82 | Yes | | 2-9-81 6-24-81 6-29-81 10-6-81 6-20-80 7-17-80 7-24-80 8-1-80 8-6-80 8-19-80 1-14-81 | Installa- tion com- plete. Testing to be com- pleted pri to startup from the lst Refuel outage per 6-29-81 le ter. Qual fication of equipment and detail operating procedures | No ior et- i- of ed |

| | | | UNIT 2 NUREG-0 | | Encl. 3 | | |
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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. NRC Req. Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
| | | | | | | to be dev loped aft system acceptanc | e- er e. |
| II.G.1 | Power supplies for pressurizer relief valves, block valves, & level indicators | Power supply from emergency buses | Fuel Load | Yes | 1-14-81 | Complete | N/A |
| II.K.1 | IE Bulletins | 5. Review ESF valves | Fuel Load | Yes | 6-20-80 | Complete | N/A |
| | | 10. Operabili'j status | Fuel Load | No | 6-20-80 | Complete | N/A |
| | | 17. Trip per low-level B/S | Fuel Load | Yes | 6-20-80 | Complete | N/A |
| 11.K.2 | Orders on B&W Plants | 13. Thermal Mechanical Report | 1-1-82 | As Required | 1-14-81 W.O.G. 81-138 dated 4-1-81 5-20-81 | See Encl 7, Item 6 | No |
| | | 17. Voiding in RCS | 1-1-82 | No | 1-14-81 | Complete See Encl 7, Item 7 | N/A |
| | | 19. Benchmark analysis seq AFW flow | 1-1-82 | No | 1-14-81 NRC letter 7-6-81 | Not required | N/A |
| II.K.3 | Final recommenda- tions, B&O task force | 1. Auto PORV isolation | lst refuel 6 mo. after staff approval | Yes | 5-26-81 | Complete | N/A |
| | | 2. Report on PORV failures | Four months prior to 1-1-81 | No | 1-14-81 WCAP-9804 3-13-81 | Complete | N/A |

| UNIT | 2 | NUREG-0737 | STATUS |
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| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | 11 Extension Requested |
|----------------------------|--------------------|---|---------------------------------|------------------------|----------------|------------------------------|---|----------------------------------|
| | | 3. Reporting SV & RV failures & challenges | Four months before 0.L. | Yes | | 1-14-81 | System in place | N/A |
| | | 5. Auto trip of RCPs | | | | | | |
| | | a. Propose mods | Prior to O.L. | No | | 1-14-81 | Suspense: Within 3 months af NRC deter mination accepta- bility of LOCA Mode See Enc?. item 15 | No ter of SB 1 7, |
| | | b. Modify | Full Power | Yes | | 1-14-81 | Suspense: Within 11 months af NRC deter mination accepta- bility of LOCA mode during an appropria outage or 1st such outage thereafter | No ter of SB 1 te |
| | | 9. PID controller | Four months before 0.L. | No | | 6-20-80 | Complete | N/A |
| | | Applicant's propose anti- cipatory trip at high power | Four months before 0.L. | Yes | | 9-16-80 | Complete | N/A |

Encl. 3

| | | | | UNIT 2 NUREG- | 0737 STATU | S | | | Enc1. 3 |
|----------------------------|--------------------|-----|---|---|------------------------|--|---|--|------------------------------|
| Clarifi- cation Item | Shortened Title | D | escription | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo I Remarks F | 12 Extension Requested |
| | | 11. | Justification use of certain PORVs | Fuel Load | No | See NUREG- 0611, Sect. 3.2.4.d. | 1-14-81 | Complete | N/A |
| | | 12. | Confirm anti- | | | | | | |
| | | | a. Propose modifica- tions | Four months before 0.L. | No | | 6-2080 | Complete | N/A |
| | | | b. Modify | Four months before 0.L. | Yes | | 6-20-80 | Complete | N/A |
| | | 17. | ECCS outages | In accord- ance with review schedule for licens- ing | As required | | 2-13-81 | Suspense: Every 5 year period after ini- tial criti- cality | No 1 |
| | | 25. | Power on pump | | | | | | |
| | | | a. Propose mods | 6 months prior to SER | No | | 1-14-81 | Complete | N/A |
| | | | b. Modifica- tions | Full Power | Yes | | 1-14-81 | Complete See Encl 7, item 8 | N/A |
| | | 30. | SB LOCA methods a. Schedule outline | In accordance with review schedule | No | | Westinghouse letter dated 11-25-81 NS-EPR-2524 | Complete | N/A |
| | | | b. Model | In accordance with review schedule | No | | Westinghouse letter dated 11-25-81 NS-EPR-2524 | Complete See Encl 7, item 9 | N/A |

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| | | | UNIT 2 NUREG-0737 STATUS | | | | | Encl. 3 |
|----------------------------|--|---|--|------------------------|---|--|--------------------------------|------------------------|
| Clarifi- cation Item | Shortened Title | Description | Implemen- tation Schedule | Tech. Spec. Req. | NRC Remarks | APCo Response Letter Date | APCo Remarks | Extension Requested |
| | | c. New analyses | In accordance with review schedule | No | | Westinghouse letter dated 11-25-81 | Complete See Encl item 9 | N/A 7, |
| | | 31. Plant- specific analysis | 1-1-83 | No | | 1-14-81 | Suspense: 1-1-83 | No |
| III.A.1.1 | Emergency preparedness, short term | Short-term improvements | Fuel Load | No | Use NUREG- 0654 until Rev. 1 is issued (due out 10/80) | 11-7-80 | Complete | N/A |
| III.A.1.2 | Upgrade emer- gency support facilities | 1. Establish TSC, OSC, EOF (interim basis) | TBD | No | | 2-13-81 | Complete | N/A |
| | | 2. Design | TBD | TBD | | 2-13-81 5-19-81 | Complete | N/A |
| | | 3. Modifications | TBD | TBD | | 2-13-81 5-19-81 | Suspense: 10-1-82 | No |
| III.A.2 | Emergency preparedness | Upgrade emer- gency plans to App E, 10 CFR 50 | Fuel Load | No | | 11-7-80 | Complete | N/A |
| | | Meteorological data | Fuel Load | No | | 2-9-81 | Suspense: 7-1-82 | No |
| III.D.1 | Primary coolant outside con- tainment | Measure leak rates & establish program to keep leakage ALARA | Full Power | Yes | | 2-9-81 9-8-81 | Complete | N/A |

| | UNIT | 2 | NUREG-0737 | STATUS |
|--|------|---|------------|--------|
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| Clarifi- | | | Implemen- | Tach | | | 14 |
|----------------|---|--|-----------------------|----------------------|-------------------------------------|-----------------|------------------------|
| cation Item | Shortened Title | Description | tation Schedule | Spec. NR Req. Rem | C APCo Response arks Letter Date | APCo Remarks | Extension Requested |
| III.D.3.3 | Inplant I ₂ radiation monitoring | Provide means to determine presence of radioiodine | Fuel Load | Yes | 1-14-81 | Complete | N/A |
| | | 2. Modifications to accurately measure radio- iodine | Prior to licensing | Yes | 1-14-81 | Complete | N/A |
| III.D.3.4 | Control-room habitability | Identify and evaluate potential haz- ards | Full Power | No | 7-17-80 | Complete | N/A |
| | | Schedule for modifications | Full Power | No | 7-17-80 | Complete | N/A |
| | | 3. Modifications | Full Power | Yes | 7-17-80 | Complete | N/A |

8.3

Enc1. 3

ENCLOSURE 4

J. M. FARLEY NUCLEAR PLANT UNIT 2

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS
J. M. FARLEY NUCLEAR PLANT UNIT 2

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED |
|--------------|---|--|---|
| I.A.1.1 | Shift Technical Advisor | Part of current FNP-2 Tech. Spec. Section 6.2.4, Figure 6.2-2, and Table 6.2-1. | |
| I.A.1.3 | Shift Manning | Part of current FNP-2 Tech. Spec. Section 6.2.2 and Table 6.2-1. | |
| I.B.1.2 | Evaluation of Organization & Management | Part of current FNP-2 Tech. Spec. Section 6.2.3. | |
| I.C.3 | Shift Super- visor Respon- sibility | Part of current FNP-2 Tech. Spec. Section 6.1.2, 6.2.2 and Table 6.2-1. | |
| I.G.1 | Training During Low-Power Testing | Not part of current FNP-2 Tech. Specs., however, training has been completed. | |
| II.B.1 | Reactor- Coolant- System Vents | Reactor Vessel Head Vents are not in FNP-2 Tech. Specs. Pressurizer Vent (PORV) is part of current FNP-2 Tech. Spec. Section 3.4.5. | Reactor Vessel Head Vents a e installed. A Tech. Spec. will be written upon NRC approval of installed system. |
| | | | |

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED |
|--------------|--|--|-----------------------------|
| II.B.3 | Postaccident Sampling | Part of current FNP-2 Tech. Spec. Section 6.8.3.e. | |
| 11.D.3 | Valve Position Indication | Part of current FNP-2 Tech. Spec. Table 3.3-11 and 4.3-7. | |
| II.E.1.2 | Auxiliary Feedwater System Initiation and Flow | Part of current FNP-2 Tech. Spec. Section 3.3.3.8 and Tables 3.3-3, 3.3-4, 3.3-5, 4.3-2, 3.3-11 and 4.3-7. | |
| II.E.3.1 | Emergency Power for Pressurizer Heaters | Part of current FNP-2 Tech. Spec. Section 3.4.4. | |
| II.E.4.2 | Containment Isolation Dependability | Part of current FNP-2 Tech. Spec. Sections 3.3.2, 3.3.3.1, 3.6.3 and 3.6.1.7. | |

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED TITLE | TECHNICAL SPECIFICATION SECTION | COMMENT/EXTENSION REQUESTED |
|--------------|--|---|---|
| II.F.1 | Accident Monitoring Instrumentation | a. Noble Gas Monitor is part of current FNP-2 Tech. Spec. Table 3.3-6 and 4.3.3. | |
| | | b. Iodine/Particulate Sampling is part of current FNP-2 Tech. Spec. Section 6.8.3.e. | |
| | | c. Containment High Range Monitor is part of current FNP-2 Tech. Spec. Table 3.3-6 and 4.3-3. | |
| | | d. Containment Pressure is not part of FNP-2 Tech. Spec. | d. Containment pressure monitor has been installed and is operational and will be included in the Unit 2 |
| | | e. Containment Water Level is part of current FNP-2 Tech. Spec. Table 3.3-11 and 4.3-7. | Upgrade of the Tech. Spec. upon receipt of the Unit 1 Tech. Spec. Upgrade. |
| | | f. Containment Hydrogen is part of current FNP-2 Tech. Spec. Section 3.6.4. | |
| II.F.2 | Instrumentation for Detection of Inadequate Core | Subcooling Meter and Incore Thermocouples are part of current FNP-2 Tech. Spec. Table 3.3-11 and 4.3-7. Reactor Vessel Water Level is not | Reactor Vessel Water Level is to be incorporated in Tech. Spec. upon appro- val by NRC of the FNP "special" level |

in FNP-2 Tech. Specs.

Reactor Vessel Water Level is to be incorporated in Tech. Spec. upon approval by NRC of the FNP "special" level system. However, the BF3 prototype system is currently under a testing program described in APCo's letter of 6-29-81. Upon completion of this test program a schedule for the Tech. Spec. will be developed.

STATUS OF

NUREG-0737 TECHNICAL SPECIFICATION REQUIREMENTS

| 0737 ITEM | SHORTENED | SPECIFICATION SECTION | COMMENT/EXTENSION | REQUESTED |
|--------------|--|--|-------------------|-----------|
| II.G.1 | Power Supplies for Pressurizer Relief Valves, Block Valves, & Level Indicators | Part of current FNP-2 Tech. Spec. Sections 3.8.2.1, 3.8.2.2, 3.8.2.3, and 3.8.2.4. | | |
| II.K.3 | Final Recom- mendations, B&O Task Force | Reporting SV & RV Failures and Challenges is part of current FNP-2 Tech. Spec. Section 6.9.1.10. 10 & 12. Anticipatory Trip on Turbine Trip is part of current FNP-2 Tech. Spec. Table 3.3-1. | | |
| III.D.1.1 | Primary Coolant Outside Contain- ment | Part of current FNP-2 Tech. Spec. Section 6.8.3.a. | | |
| III.D.3.3 | Inplant I ₂ Radiation Monitoring | Part of current FNP-2 Tech. Spec. Section 6.8.3.b. | | |
| III.D.3.4 | Control-Room Habitability | Part of current FNP-2 Tech. Spec. Sections 3.3.3.6 and 3.7.7. | | |

ENCLOSURE 5

J. M. FARLEY NUCLEAR PLANT UNIT 2

STATUS OF

FULL POWER LICENSE REQUIREMENTS

Encl. 5

UNIT 2 LICENSE STATUS

| License <u>Reference</u> | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|-----------------------------|---|--|---|--|--|------------------------|
| 2.C.(6) | - | Appendix R, 10 CFR 50.48 | Per 3-19-81 APCo letter | 3-19-81 | See Enclosure 6, item 9 | Yes |
| 2.C.(7) | - | 90 day report | 6-29-81 | 6-25-81 | Complete | N/A |
| | III.A.1.2 | Upgrade Emergency Support | 5-17-81 | 5-19-81 | Conceptual design | N/A |
| | | racificies | 10-1-82 | 5-19-81 | Suspense: 10-1-82 | No |
| | III.A.2 | Long Term Emergency Pre- paredness | 3-1-82 7-1-82 9-1-82 6-1-83 | 1-14-81 2-9-81 | Suspense: 3-1-82 Suspense: 7-1-82 Suspense: 9-1-82 Suspense: 6-1-83 | No No No No |
| | Appendix B E Preparedness Section B | mergency Evaluation Report Onsite Emergency Organiza- tion | 9-1-81 | - | Complete | N/A |
| | Section E | Notification Methods and Procedures | 6-1-81 8-31-81 11-1-81 | 5-21-81 7-23-81, 10-20-81 - | Complete | N/A |
| | | | 6-29-81 10-1-81 11-1-81 12-15-81 | 5-21-81, 2-23-81 9-4-81 10-20-81 | Complete | N/A |
| 2.C.(8) | - | Complete modifications to both service water loops as one-time only exemption of T.S. 3.7.4 | 6-29-81 | 6-29-81 | Complete | N/A |
| 2.C.(9)(a) | - | Provide results of seven augmented low power tests | Prior to exceeding 5% Power | 5-18-81 | Complete | N/A |
| | | | | | | |

N/A - There are no further requirements for this item.
 No - Remaining requirements are expected to be completed on schedule.
 Yes - An extension for this item was requested in the letter listed below.

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UNIT 2 LICENSE STATUS

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| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------|-------------------------|---|--|---|---|------------------------|
| 2.C.(9)(b) | | Provide results of test of natural circulation with boron mixing | Within 60 days after operation for 25,000 MW(e) days | 9-16-81 NRC letter dated 10-20-81 | Now part of 2.C.(12)(c) test to be run, if required, at the 1st Refuel Outage | N/A |
| 2.C.(10) | II.B.3 | Make fully operational post- accident sampling system | Prior to exceeding 5% Power | 1-14-81 | Complete | N/A |
| 2.C.(11) | II.B.4 | Complete training to miti- gate core damage | Prior to exceeding 5% Power | 1-14-81 | Complete | N/A |
| 2.C.(12)(a) | - | Perform tests to demonstrate manual operation of an at- mospheric steam dump valve | Prior to exceeding 5% Power | - | Complete | N/A |
| 2.C.(12)(b) | II.B.1 | Make provisions or modifi- cations to assure the safety grade backup means of the reactor coolant system depressurization is in accordance with require- ments | Prior to Startup following lst Refuel | - | See Enclosure 7, item 3 | No |
| 2.C.(12)(c) | - | Provide natural circulation cooldown procedures | Prior to Startup following 1st Refuel | 9-16-81 | Suspense: Prior to Startup follow- ing 1st Refuel, if required. | No |
| 2.C.(13) | - | Revise procedures, modify and test reset circuits for containment air mixing fans, containment purge isolation valves and auxiliary feed- water pump discharge valve | Prior to exceeding 5% Power | 9-8-80 9-12-80 9-29-80 4-27-81 | Complete | N/A |

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| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------|-------------------------|---|--|------------------------------|---|------------------------|
| 2.C.(14) | - | Demonstrate the operability of TS Table 3.3-12 fire detectors and TS Table 3.4-1 reactor coolant system isolation valves | Prior to exceeding 5% Power | • | Complete | N/A |
| 2.C.(15) | II.F.2 | Complete modifications to subcooling monitor system | 5-31-81 | | Complete | N/A |
| 2.C.(16) | | Provide response to I.E. Bulletin 80-11, including reevaluation report | 5-17-81 | 5-12-81 | Complete no later than 1st Refueling | N/A |
| 2.C.(17) | II.E.4.2 | Provide design of modified containment vent and purge system to reduce use of 18" purge valves | 10-1-81 | 9-30-81 10-30-81 | Complete | N/A |
| | | Install modified containment vent and purge system | Prior to Startup following lst Refuel | 9-30-81 10-30-81 | Suspense: Prior to Startup follow- ing 1st Refuel | No |
| 2.C.(18)(a) | - | Complete and have auditable records available at a central location to docu- ment compliance with envi- ronmental qualification requirements for Class IE equipment | 6-30-82 | - | Suspense: 6-30-82 | No |
| 2.C.(18)(b)(i) | - | Correct or commit to correct noncompliance with NUREG-0588 equipment quali- fication safety evaluation | 7-14-81 | 7-1-81 | See APCo response 7-1-81 | N/A |

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UNIT 2 LICENSE STATUS

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| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------|-------------------------|---|--|------------------------------|---|------------------------|
| 2.C.(18)(b)(ii) | | Commit to corrective actions which will result in documen- tation of compliance of applicable equipment with NUREG-0588. | 6-30-82 | | Additional informa- tion will be pro- vided in the 12-28-81 submittal | No |
| 2.C.(18)(c) | - | Qualify all safety-related electrical equipment in the facility | 6-30-82 | - | Suspense: 6-30-82 | No |
| 2.C.(19)(a) | - | Provide additional evalua- tions of the Westinghouse fuel performance code. | Prior to Startup following lst Refuel | | Suspense: Prior to Startup follow- ing 1st Refuel | No |
| 2.C.(19)(b) | | Complete modifications of primary and backup circuit protection devices in con- tainment electrical pene- tration circuits | Prior to Startup following lst Refuel | | Suspense: Prior to Startup following 1st 1st Refuel (Seis- mic testing required prior to manufacturer's shipment 9/82 for 10/82 outage) | No |
| 2.C.(19)(c) | - | Submit system final design and implementation schedule of the modification of the lubrication system of the two Fairbanks-Morse opposed piston diesel generators | 9-30-81 | 9-21-81 | Complete | N/A |
| | | Modify lubrication system of the two Fairbanks-Morse opposed-piston diesel generators | Prior to Startup following lst Refuel | 9-21-81 | Suspense: Prior to Startup following 1st Refuel | No |

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| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------|-------------------------|--|--|-----------------------------------|---|-----------------------------------|
| 2.C.(19)(d) | - | Inspect main steam turbine for low pressure rotor disc cracking or replace rotors | Prior to Startup following lst Refuel | 2-1-82 | See APCo letter letter 2-1-82 | Yes |
| 2.C.(20) | - | Provide a schedu?e for bringing the facility into compliance with Rev. 2 to R. G. 1.97 | 4-30-81 | 3-30-81 11-16-81 | See 11-16-81 APCo letter | N/A |
| 2.C.(21)(a) | I.C.1 | Complete upgrading of emer- gency procedures and asso- ciated operator training | Prior to Startup following lst Refuel after 1-1-82 | - | See Enclosure 7, item 1 | No |
| 2.C.(21)(b) | II.B.1 | Submit design description and operating procedures for reactor coolant system vents | 7-1-81 | 6-25-81 WOG letter 11-30-81 | Complete | N/A |
| | | Complete installation of reactor coolant system vents | 7-1-82 | - | Complete | N/A |
| 2.C.(21)(c) | II.B.2 | Complete modifications to assure access to vital areas and protection of safety equipment following a degraded core accident | 4-1-82 | | Suspense: 4-1-82 | Yes See Encl. 7, item 14 |
| 2.C.(21)(d)(1) | II.D.1 | Report qualification of relief valves and piping | 10-1-81 | 6-25-81 9-30-81 | APCo is a part of EPRI program and will provide additional infor- mation as it be- comes available | N/A |

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UNIT 2 LICENSE STATUS

| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|----------------------|-------------------------|---|--|------------------------------|---|------------------------|
| 2.C.(21)(d)(2) | II.D.1 | Report qualification of block valves | 7-1-82 | | Suspense: 7-1-82 APCo is partici- pating in EPRI submittal | No |
| 2.C.(21)(e) | II.E.1.2 | Submit design of modifica- tions to the control and protection circuits for the auxiliary feedwater systems | 7-1-81 | 7-1-81 | Complete | N/A |
| | | Modify control and protec- tion circuits for auxiliary feedwater system | Prior to Startup following lst Refuel | | Suspense: Prior to Startup follow- ing 1st Refuel | No |
| 2.C.(21)(f)(1) | II.F.1 | Install noble gas effluent monitors | 1-1-82 | - | Complete See Enclosure 7, item 5 | N/A |
| 2.C.(21)(f)(2) | II.F.1 | Install capability for con- tinuous sampling of plant gas effluents | Prior to exceeding 5% Power | - | Complete See Enslosure 7, item 5 | N/A |
| 2.C.(21)(f)(3) | II.F.1 | Install high-range radio- activity monitors in con- tainment | 1-1-82 | - | Complete | N/A |
| 2.C.(21)(f)(4) | II.F.1 | Provide a description of containment pressure in- struments | 6-1-81 | 6-1-81 | Complete | N/A |
| | | Install containment pressure instruments | 1-1-82 | - | Complete | N/A |

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| Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response | APCo | Extension |
|----------------|-------------------------|---|---------------------|---------------|----------------------------|-----------|
| 2.C(21)(f)(5) | II.F.1 | Provide a description | Juc Duce | Letter Date | Remarks | Requested |
| | | containment water level measurement system | 6-1-81 | 6-1-81 | Complete | N/A |
| | | Install containment water level system | 1-1-82 | - | Complete | N/A |
| 2.C.(21)(f)(6) | II.F.1 | Provide a description of the installed hydrogen indica-tion monitors | 6-1-81 | 6-1-81 | Complete | N/A |
| | | Make modifications to the installed hydrogen indica- tion monitors, if required | 1-1-82 | - | Not required | N/A |
| 2.C.(21)(g)(1) | II.F.2 | Provide detailed design in- formation for proposed reactor vessel water level instrument | 7-1-81 | 6-29-81 | Complete | N/A |
| 2.C.(21)(g)(2) | II.F.2 | Provide results of FNP-1 tests of proposed reactor vessel water level instru- ment | 7-1-81 | 6-24-81 | Complete | N/A |
| 2.C.(21)(g)(3) | II.F.2 | Provide planned program to complete reactor vessel water level instrument development, and feasibility data thereto | 1-1-82 | 6-29-81 | Complete | N/A |
| .C.(21)(h)(1) | II.K.2.13 | Submit detailed analysis of 1 thermal mechanical conditions in reactor vessel during recovery from small LOCA with an extended loss of all feedwater | -1-82 | 5-20-81 | See Enclosure 7, item 6 | No |

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| License Reference | NUREG-0737 Reference | Subject | License Jue Date | APCo Response Letter Date | APCo Remarks | Extension Requested |
|------------------------------|-------------------------|---|---|------------------------------|---|------------------------|
| 2.C.(21)(h)(2) | II.K.2.17 | Provide analysis of poten- tial for voiding in reactor coolant system | 1-1-82 | - | See Enclosure 7, item 7 | N/A |
| 2.C.(21)(h)(3) | II.K.2.19 | Provide benchmark analysis of sequential feedwater flow following loss of main feed- water | 1-1-82 | NRC letter 7-6-81 | Not required | N/A |
| 2.C.(21)(i)(1) (i) & (ii) | II.K.3.1 II.K.3.2 | Provide information required by NUREG-0737 if automatic PORV isolation system is required as the result of safety examination | 7-1-81 | 5-26-81 | Complete | N/A |
| 2.C.(21)(i)(1) (iii) | II.K.3.1 | Complete installation and testing of modified automa- tic isolation system | Prior to Startup following 1st Refuel 6 months after NRC design | 5-26-81 | Not required | N/A |
| 2.C.(21)(i)(2) (i) | II.K.3.5 | Provide evaluation or design modification for tripping of reactor coolant pumps in the event of a small break LOCA | Within 3 months after NRC determination of accepta- bility of small break LOCA model | - 1 | Suspense: Within 3 months after NRC determination of acceptability of small break LOCA model. See Enclosure 7, item 15 | No |

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| License Reference | NUREG-0737 Reference | Subject | License Due Date | APCo Response Letter Date | APCo <u>Remarks</u> | Extension Requested |
|------------------------|-----------------------------|---|---|------------------------------|--|------------------------|
| 2.C.(21)(i)(2) (ii) | II.K.3.5 | Complete plant modifications for small break LOCA trip- ing of reactor coolant pumps | Within 11 months after NRC determi- nation of acceptability of small brea LOCA model during an appropriate outage or first such outage there- after | - | Suspense: Within 11 months after NRC determination of acceptability of small break LOCA model during an appropriate outage or first such outage there- after. | No |
| 2.C.(21)(i)(3) (i) | 11 . K . 3.25 | Submit results of analysis to determine consequences of loss of cooling water to the reactor coolant pump seal coolers and describe any modifications necessary | 1-1-82 | - | See Enclosure 7, item 8 | N/A |
| 2.C.(21)(i)(3) (ii) | 11.K.3.25 | Complete any necessary modi- fications to the reactor coolant pump seal coolers | 7-1-82 | - | See Enclosure 7, item 8 | N/A |
| 2.C.(21)(i)(4) (i) | JI.K.3.30 | Submit revised small break LOCA model to account for recent experimental data | 1-1-82 | - | Complete See Enclosure 7, îtem 9 | N/A |
| 2.C.(21)(i)(4) (ii) | II.K.3.31 | Submit results of plant specific calculations using NRC approved revised small break LOC4 model | 1-1-83 | - | Suspense: 1-1-83 | No |

Encl. 5

| License | NUREG-0737 | Subject | License | APCo Response | APCo | Extension |
|-----------|------------|--|----------|---------------|----------|-----------|
| Reference | Reference | | Due Date | Letter Date | Remarks | Requested |
| 2.(D) | - | Implement commitments iden- tified in Chapter 15, items 15.1.d, f and g of Physical Security Plan | 8-1-81 | 7-31-81 | Complete | N/A |

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ENCLOSURE 6

J. M. FARLEY NUCLEAR PLANT

STATUS OF

UNITS 1 & 2 OTHER REQUIREMENTS

Encl. 6

OTHER LICENSING REQUIREMENTS*

| Item No. | Subject | NRC Due Date | APCo Response Date | Remarks | Extension Requested |
|----------|---|--|-----------------------|---|------------------------|
| 1. | Provide 90 day response on USI A-44 (Station Blackout) (Unit 1 only) | 1-1-82 | 12-7-81 | Complete | N/A |
| 2. | Complete an evaluation to extend the qualified life of equipment (Environ- mental Qualification) | 1-1-82 | - | To be submitted under separate letter on 12-28-81 | No |
| 3. | Revise plant procedures (Control of Heavy Loads, NUREG-0612) | 10-1-81 | - | See Encl. 7, item 10 | No |
| 4. | Correct remaining fire damper defi- ciencies | Prior to Startup following lst Refuel Unit 2 | - | Suspense: Prior to Startup fol- lowing 1st refuel | No |
| | | Prior to Startup following 3rd Refuel Unit 1 | - | Suspense: Prior to Startup fol- lowing 3rd Refuel See Encl. 7, item 11; I&E Audit Report dated 4-6-81 | No |
| 5. | Conduct UT (Flow splitters) inspection (Unit 1 only) | Prior to Startup following 3rd Refuel | - | Complete | N/A |
| 6. | Complete Hot Shutdown panel modifica- tions (Unit 2 only) | Prior to Startup following 1st Refuel | 9-30-80 | Design complete Suspense: Prior to Startup fol- lowica 1st Refuel | No |

* Unless otherwise noted, all items refer to both Units 1 and 2.

Enc1. 6

OTHER LICENSING REQUIREMENTS*

| Item No. | Subject | NRC Due Date | APCo Response Date | Remarks | Extension Requested |
|----------|--|--|---|---|----------------------------|
| 7. | Oil Collection for Reactor Coolant Pump | Prior to Startup following 3rd Refuel for Unit 1 | 3-19-81 | See Encl. 7, item 13 | See Encl. 7, item 13 |
| | | Prior to Startup following lst Refuel for Unit 2 | 3-19-81 | Suspense: Prior to Startup following 1st Refuel Set Encl. 7, item 13 | No |
| 8. | Radiological Emergency Response (Class A Model) | 1-1-82 | 12-29-79 | See Encl. 7, item 12 | N/A |
| 9. | Fire Protection Upgrade (10 CFR 50, Appendix R) | 3-1-82 for Unit 2 | 3-19-81 | Plan and schedule provided in referenced letter | Yes 3-19-81 |
| | | 10-1-82 for Unit 1 | 3-19-81 | Plan and schedule provided in referenced letter | Yes 3-19-81 |
| 10. | RETS (10 CFR 50, Appendix I) Upgrade and Snubber Upgrade Unit 2 | Unit 2 T.S. Upgrade Package | 30 days within receipt of issuance of Unit 1 Upgrade package currently scheduled for December, 1981. | To be included as part of the Unit 2 Upgrade Package | N/A |
| 11. | Unit 2 T.S. Upgrade | | 30 days within receipt of issuance of Unit 1 Upgrade currently scheduled for December, 1981. | This package will be identical. (less specific unit differences from the Unit 1 Upgrade) | N/A |

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Encl. 6

OTHER LICENSING REQUIREMENTS*

| Item No. | Subject | NRC Due Date | APCo Response Date | Remarks | Extension Requested |
|----------|--|--------------------|---|---|------------------------|
| 12. | Station Blackout Training | W.O.G. 11-30-81 | See Enclosure 7, item 1 | This training is a part of the procedure upgrade required by NUREG- 0737, Item I.C.1 | N/A |
| 13. | D.G. T.S. Upgrade (Unit 2) | | 10-28-81 11-6-81 | Same as section in Unit 2 T.S. Upgrade except for 26 second delay in startup of D/G 2C. Scheduled to be issued by NRC in January, 1982. | N/A |
| 14. | F _Q Upgrade | | 11-16-81 11-23-81 12-14-81 | To be handled as part of the Unit 2 T.S. Upgrade package. | N/A |
| 15. | Masonry Walls (IEB 80-11) | N/A | Unit 1 5-22-81 Unit 2 5-12-81 | None | N/A |
| 16. | MSL Break With Continued AFW Addition (IEB 80-04) | None | 5-8-80 | None | N/A |
| 17. | HELB (IEN 79-22) | None | Unit 1 10-5-79 Unit 2 6-5-81 NRC 9-1-81 letter accepting position | None | N/A |
| 18. | Seismic Qualification of AFW System | None | 10-9-81 | Complete | N/A |

Encl. 6

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OTHER LICENSING REQUIREMENTS*

| Item No. | Subject | NRC Due Date | APCo Response Date | Remarks | Extension Requested |
|----------|--|----------------------|--------------------------|---|------------------------|
| 19. | Natural Circulation Cooldown (Generic Letter 81-21 St. Lucie Event) | None | 11-13-81 | Procedures to be developed 90 days after receipt of W.O.G. guidelines | N/A |
| 20. | NRC Approval of IST Program For Unit 2 | None | Scheduled for 3-15-82 | APCo is develop- ing a revised program scheduled for submitted to the NRC by 3-15-82 | N/A |
| 21. | Approval of the Unit 1 and 2 Technical Specifications For Containment Purge Valves | NRC letter 8-5-81 | 9-23-81 | Tech. Spec. will be revised just prior to instal- lation of the 8 inch purge valves. | |

EFCLOSURE 7

J. M. FARLEY NUCLEAR PLANT

INITIAL SUBMITTALS AND EXTENSION BASIS DOCUMENTATION

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| Item No | Subject | Enclosure Number | Item/ Paragraph |
|------------|---|---------------------|---|
| 1. | I.C.1, Guidance for the evaluation and development of procedures for transients and accidents | 1 3 5 | 1.C.1 I.C.1 2.C.(21)(a) |
| 2. | I.D.1, Control room design reviews | 1 3 | I.D.1 I.D.1 |
| 3. | II.B.1, Reactor Coolant System vents | 1 3 5 | II.B.1 II.B.1 2.C.(12)(b) |
| 4. | II.E.1.2, Auxiliary Feedwater Initiative and Indication | 1 | II.E.1.2 |
| 5. | II.F.1, Accident Monitoring Instru- ments | 1 3 5 | II.F.1 II.F.1 2.C.(21)(f)(1) |
| 6. | II.K.2.13, Thermal Mechanical Report | 1 3 5 | II.K.2.13 II.K.2.13 2.C.(21)(h)(1) |
| 7. | II.K.2.17, Voiding in RCS | 1 3 5 | II.K.2.17 II.K.2.17 2.C.(21)(h)(2) |
| 8. | II.K.3.25, Loss of AC on Pump Seals | 1 3 5 | II.K.3.25 II.K.3.25 2.C.(21)(i)(3)(i) |
| 9. | II.K.3.30, Revise SBLOCA Methods | 1 3 5 | II.K.3.30 II.K.3.30 2.C.(21)(i)(4)(i) |
| 10. | Control of Heavy Loads | 6 | 3 |
| 11. | Fire Damper Correction | 6 | 4 |
| 12. | Radiological Emergency Response | 6 | 8 |
| 13. | Reactor Coolant Pump, Oil Collection System Upgrade per 10 CFR 50, Appendix | ۶ R | 7 |
| 14. | II.B.2, Plant Shielding | 1 3 5 | II.B.2 II.B.2 2.C.(21)(c) |
| 15. | II.K.3.5, Auto Trip of RCP during LOCA | 1 3 5 | II.K.3.5 II.K.3.5 2.C(21)(i)(2)(i) |

Item 1. I.C.1 GUIDANCE FOR THE EVALUATION AND DEVELOPMENT OF PROCEDURES FOR TRANSIENTS AND ACCIDENTS [Enclosures 1 and 3, item I.C.1; Enclosure 5, para. 2.C.(21)(a)]

Requirement

 By letter to the NRC dated January 14, 1981, Alabama Power Company made the following commitment on this subject related to the Farley Nuclear Plant.

The Westinghouse Owners Group submitted a detailed description of its program to comply with the requirements of Item I.C.1 on December 15, 1980 (WOG Letter 80-179). The program identified previous owners group submittals to the NRC, which we believe will comprise the bulk of the response. The additional effort required to obtain full compliance with this item, as discussed with the NRC on November 12, 1980, together with a schedule for completion, was also identified in the December 15, 1980, submittal. If additional guidelines are developed, the Farley Nuclear Plant will utilize these guidelines to further upgrade appropriate procedures and provide the associated operator training by the first refueling outage for each unit after January 1, 1982.

 Unit 2 License Item 2.C.(21)(a) states that prior to startup following the first refueling after January 1, 1982, Alabama Power Company should complete the upgrading of emergency procedures and associated operator training.

Response

1. Alabama Power Company, as a member of the Westinghouse Owners Group, is participating in a significant upgrade of emergency procedures with accompanying appropriate operator training as a result of this requirement. The Wastinghouse Owners Group submitted a detailed description of the revised Emergency Response Guideline Program (ERG) to Item 1.C.1 of NUREG-0737 in OG-61 dated July 7, 1981. The Westinghouse Owners Group transmitted all completed portions of the ERG set to the NRC for their review in letter OG-64 dated November 30, 1981. Alabama Power Company is confident that the materials in the ERG set which has been transmitted to the NRC are sufficient to satisfy the requirements of NUREG-0737, I.C.1.

The upgrade of plant specific procedures must be coordinated with other NRC criteria. These criteria include the main control room review (NUREG-0700 and 801), emergency response upgrade (NUREG 835, 696, 814, R. G. 1.97), and operating procedures (NUREG-0799). Some of these NUREGs are in draft form and have been issued for comment. To enable Alabama Power Company to implement an integrated system which addresses all criteria, such new criteria must be finalized by the NRC and included in the integrated efforts associated with the main control room review and emergency response upgrade (SPDS).

Item 1. I.C.1 (Cont'd)

Alabama Power Company received the approved Westinghouse Owners Group guidelines in December, 1981. These guidelines will be utilized to upgrade plant specific procedures and provide associated operator training by the fourth refueling outage for Unit 1 (scheduled for the spring of 1983), and the first refueling outage for Unit 2 (scheduled for November, 1982).

The above proposed schedule is contingent upon timely finalization of new NRC criteria associated with the main control room review, emergency response upgrade and operating procedures.

Item 2. I.D.1 CONTROL ROOM DESIGN REVIEWS [Enclosures 1 and 3, item I.D.1]

Requirement

By letter of January 14, 1981, Alabama Power Company made the following commitment on this item.

Alabama Power Company intends to implement similar commitments described in the July 17, 1980 letter for Unit 1 prior to return to power after the current refueling outage (second) except for denoting normal, alert, and alarm ranges on other significant main control room meters (other than those described in emergency procedures) which will be completed by the end of the third refueling outage. Alabama Power Company will address the long term control room design review after issuance of NUREG 0700. Alabama Power Company intends to implement the commitments described in the July 17, 1980 letter for Unit 2 prior to exceeding 5% power except for denoting normal, alert, and alarm ranges on other significant main control room meters (other than those described in emergency procedures) which will be completed by the end of the first refueling outage.

Response

Alabama Power Company has completed the Unit 1 and 2 modifications associated with the commitments described in the July 17, 1980 letter. Denotation of the Unit 1 normal, alert, and alarm ranges has been completed on the other significant main control room meters and will be completed for Unit 2 by the end of the first refueling outage scheduled to begin in November, 1982.

NUREG 0700 was issued in September, 1981 and Alabama Power Company is in the process of reviewing this NUREG and developing an implementation plan. Subsequently, additional draft NUREGs which affect the main control room review have been issued for comment. These draft NUREGs include NUREG-0801, "Evaluation Criteria for Detailed Concrol Room Design Review"; NUREG-0799, "Draft Criteria for Preparation of Emergency Operating Procedures"; NUREG-0814, "Methodology for Evaluation of Emergency Response Facilities"; and NUREG-0835, "Human Factors Acceptance Criteria for the Safety Parameter Display System".

To enable Alabama Power Company to perform the main control room review in a manner that will be consistent with criteria currently in review (draft NUREGS), such new criteria must be finalized by the NRC and included in the Farley main control room review plan.

Alabama Power Company will provide an implementation plan and schedule for the main control room review upon finalization of the NRC criteria. Item 3. II.B.1 REACTOR COOLANT SYSTEM VENTS [Enclosures 1 and 3, item II.B.1; Enclosure 5, para 2.C.(12)(b)]

Requirement

1. By letter dated January 14, 1981, Alabama Power Company committed to the actions described below for this item.

The Westinghouse Owners Group, of which Alabama Power Company is a member, is developing generic procedure guidelines for the use of the reactor vessel head vent system which will be incorporated into the Farley Plant procedures when the Reactor Coolant System Vent System is operational and approved by the NRC.

The additional displays and controls added to the control room as a result of this requirement will be considered as part of the long-term human-factors analysis and will include:

- a) the use of this information by an operator during both normal and abnormal plant conditions
- b) integration into emergency procedures
- c) integration into operator training, and
- d) other alarms during emergency and need for prioritization of alarms.
- 2. Unit 2 License Item (2.C.(12)(b)) states that prior to startup following the first refueling, Alabama Power Company shall make provisions (or modifications) as necessary to assure that the safety grade backup means of reactor coolant system depressurization is in accordance with the requirements of Table 1 in Branch Technical Position RSB 5-1, Rev. 1.

Response

 This system has been installed and tested for Units 1 and 2. This system will not be placed in an operational status on either unit until accepted by the NRC.

The Westinghouse Owners Group has developed generic procedures guidelines for the use of the reactor vessel head vent system. Alabama Power Company will develop plant specific procedures based on the generic guidelines as described in response to item I.C.1. The additional displays and controls added to the control room as a result of this requirement will be included in the main control room review as described in response to item I.D.1.

 Alabama Power Company is reviewing, with Westinghouse Electric Corporation, the available options that address BTP-RSB 5-1, Rev.
 If conditions arise that prevent implementation of an acceptable safety grade backup depressurization system, Alabama Power Company will notify the NRC Staff. Item 4. II.E.1.2 AUXILIARY FEEDWATER INITIATION AND INDICATION [Enclosure 1. item II.E.1.2]

Requirement

By letter dated July 1, 1981, from Alabama Power to the NRC:

In accordance with my letter dated December 4, 1980 to Mr. R. L. Tedesco and as required by Farley Nuclear Plant Unit 2 Operating License NPF-8 Section 2.C.(21)(e), Alabama Power Company submits for NRC review the attached description of proposed modifications to the Farley Nuclear Plant Auxiliary Feedwater control and protection design. In order for Alabama Power Company to implement these changes on Farley Unit 1 during the third refueling outage, your concurrence is requested no later than August 1, 1981. These changes have been agreed to in principle as a part of the Farley Unit 2 full power licensing process.

Response

At the time that Alabama Power Company committed to implementing the proposed modifications on Farley Unit 1 during the third refueling outage, that outage was scheduled to begin on or about March 12, 1982. Main generator problems forced Farley Unit 1 into an unexpected outage in early September, 1981. Because of the extensive repairs required to restore the generator, a decision was made to refuel the reactor early thereby avoiding a refueling during the summer of 1982.

Design and procurement were criginally scheduled to support the outage beginning in March, 1982. All NRC required modifications to the Auxiliary Feedwater have been completed except those identified in Alabama Power Company's December 4, 1980 letter. These requirements will provide the operator with more positive control of the flow control valves as described in this letter. Design to implement requirements in the December 4, 1980 letter was approved by the NRC in August, 1981. Procurement was delayed pending NRC approval to preclude unnecessary purchase of equipment. At the time of the unexpected outage a review was made to determine if long lead time items could be procurred in time to support the revised outage schedule. Procurement lead time exceeded the original outage duration schedule. Subsequent to this review the outage duration expanded in a fragmented fashion as additional problems were discovered on the turbine generator. In addition, the proposed design change is a major modification affecting a safety related system and would require significant review time by the plant staff to plan the implementation and verify that other safety related systems would not be impacted. Outage manpower has already been allocated and any additions impact already scheduled work.

Alabama Power Company is continuing to review the impact of implementing these modifications and will make every effort to complete this item during the current outage but in any event all modifications will be completed by the end of the fourth refueling outage currently scheduled for February of 1983. However, if any scheduled outage occurs of sufficient duration, Alabama Power Company will implement the auxiliary feedwater modification prior to startup following the fourth refueling outage.

- Item 5. II.F.1 ADDITIONAL ACCIDENT MONITORING INSTRUMENTATION
 [Enclosures 1 and 3, item II.F.1; Enclosure 5,
 para 2.C.(21)(f)(1)]
 - 1. Noble Gas Effluent Monitors

Requirement

By letter dated January 14, 1981 Alabama Power Company committed to develop procedures for operating, calibration, and dissemination of release rate information asociated with noble gas effluent monitors.

Response

The above procedures have been written for the Sping-4 monitor and are being written for the condenser air removal exhaust and steam generator safety relief valves and atmospheric relief discharge monitors. The Unit 1 and 2 procedures currently being written are scheduled to be approved by January 1, 1982. Procedures to disseminate release rate information have been approved for Units 1 and 2.

2. Sampling and Analysis of Plant Effluents

Requirement

By letter dated January 14, 1981 Alabama Power Company committed that for vent stack effluents there will be a Victoreen vacuum pump with charcoal filters what will allow the Chemistry and Health Physics Group to draw 15 minute iodine and particulate samples to be analyzed in the laboratory.

Response

A vacuum pump has been installed for Units 1 and 2 that will allow 15 minute samples to be drawn and analyzed in the laboratory.

Item 6. II.K.2.13, THERMAL MECHANICAL REPORT [Enclosures 1 and 3, item II.K.2.13; Enclosure 5, para 2.C.(21)(h)(1)]

Requirement

 By letter to the NRC dated January 14, 1981, Alabama Power Company made the following commitment on this subject related to the Farley Nuclear Plant.

To completely address the NRC requirements for a detailed analysis of the thermal-mechanical conditions existing in the reactor vessel during recovery from small breaks with an extended loss of all feedwater. Alabama Power Company, as a member of the Westinghouse Owners Group, is participating in a program consisting of analysis for generic Westinghouse PWR plant groupings. The program will be completed and documented to the NRC by January 1,1982.

Following completion of this generic program, additional plant specific analysis, if required, will be provided. A schedule for the plant specific analysis will be determined based on the results of the generic analysis.

- Unit 2 License item 2.C.(21)(h)(1) states that prior to January 1, 1982, Alabama Power Company should submit a detailed analysis of thermal-mechanical conditions in the reactor vessel during recovery from small LOCA with an extended loss of all feedwater.
- By letter to the NRC dated May 20, 1981, Alabama Power Company made the following commitment on this subject related to the Farley Nuclear Plant.

Verify by January 1, 1982, completion of W.O.G. program and determine if future additional plant specific analysis ...d/or remedial actions are required.

Response

Westinghouse (in support of the Westinghouse Owners Group) is performing an analysis for generic Westinghouse plant groupings to address this issue which will be submitted to the NRC by the end of 1981. This generic study will be applicable to Farley Nuclear Plant. Item 7. II.K.2.17, VOIDING IN RCS [Enclosures 1 and 3, item II.K.2.17, Enclosure 5, para 2.C.(21)(h)(2)]

Requirement

 By letter to the NRC dated January 14, 1981, Alabama Power Company made the following commitment on this subject related to the Farley Nuclear Plant.

The Westinghouse Owners Group, of which Alabama Power Company is a member, is currently addressing the potential for void formation in the Reactor Coolant System (RCS) during natural circulation cooldown conditions, as described in Westinghouse letter NS-TMA-2298 (T. M. Anderson of Westinghouse to P. S. Check of the NRC). We believe the results of this effort will fully address the NRC requirement for analysis to determine the potential for voiding in the RCS during anticipated transients. A report describing the results of this effort will be provided to the NRC by January 1, 1982.

 Unit 2 License item 2.C.(21)(h)(2) states that prior to January 1, 1982, Alabama Power Company should submit an analysis of the potential for voiding in the reactor coolant system during anticipated transients.

Response

Westinghouse (in support of the Westinghouse Owners Group) has performed a study which addresses the potential for void formation in Westinghouse designed nuclear steam supply systems during natural circulation cooldown/depressurization transients. This study has been submitted to the NRC by the Westinghouse Owners Group letter OG-57, dated 4-20-81, Jurgenson to Check and is applicable to the Farley Nuclear Plant.

In addition, the Westinghouse Owners Group has developed a natural circulation cooldown guideline that takes the results of the study into account so as to preclude void formation in the upper head region during natural circulation cooldown/depressurization transients, and specifies those conditions under which upper head voiding may occur. These Westinghouse Owners Group generic guidelines have been submitted to the NRC (letter OG-64, dated 11-30-81, Jurgensen to Eisenhut). The generic guidance developed by the Westinghouse Owners Group will be utilized in the implementation of the Farley Nuclear Plant specific operating procedures as part of Alabama Power Company's NUREG-0737, item I.C.1 effort.

Item 8. II.K.3.25, LOSS OF AC ON PUMP SEALS [Enclosures 1 and 3, item II.K.3.25; Enclosure 5, para 2.C.(21)(i)(3)(i)]

Requirement

- NUREG-0737 require the submission of the evaluation and proposed modifications for the above item by January 1, 1982.
- Unit 2 License item 2.C.(21)(i)(3)(i) states that prior to January 1, 1982, Alabama Power Company should submit results of analyses or experiments to determine consequences of a loss of cooling water to the reactor coolant pump seal coolers and decribe any modifications found necessary.

Response

This item requires that the consequences of a loss of RCP seal cooling due to a loss of AC power (defined as loss of offsite power) for at least 2 hours be demonstrated.

During normal operation, seal injection flow from the chemical and volume control system is provided to cool the RCP seals and the component cooling water system provides flow to the thermal barrier heat exchanger to limit the heat transfer from the reactor coolant to the RCP internals. In the event of loss of offsite power the RCP motor is deenergized and both of these cooling supplies are terminated; however, the diesel generators are automatically started and either seal injection flow or component cooling water to the thermal barrier heat exchanger is automatically restored within seconds. Either of these cooling supplies is adequate to provide seal cooling and prevent seal failure due to loss of seal cooling during a loss of offsite power for at least 2 hours. Item 9. II.K.3.30, REVISE SBLOCA METHODS [Enclosures 1 and 3, item II.K.3.30; Enclosure 5, para 2.C.(21)(i)(4)(i)]

Requirement

- NUREG-0737 requires the submission of additional information on the subject item by January 1, 1982.
- Unit 2 License item 2.C.(21)(i)(4)(ii) states that prior to January 1, 1982, Alabama Power Company should submit to the NRC a revised model to account for recent experimental data.

Response

This item requires that the analysis methods used by NSSS vendors and/or fuel suppliers for small-break LOCA analysis for compliance with Appendix K to 10 CFR Part 50 be revised, documented, and submitted for NRC approval.

Westinghouse feels very strongly and Alabama Power Company agrees that the small-break LOCA analysis model currently approved by the NRC for use on Farley Nuclear Plant is conservative and in conformance with Appendix K to 10 CFR Part 50. However, Westinghouse believes that improvement in the realism of small-break calculations is a worthwhile effort and has committed to revise its small-break LOCA analysis model to address NRC concerns (e.g., NUREG-0611, NUREG-0623, etc.). This revised Westinghouse model is currently scheduled for submittal to the NRC by April 1, 1982 as documented in Westinghouse letter NS-EPR-2524, dated 11-25-81, Rahe to Eisenhut. Item 10. CONTROL OF HEAVY LOADS [Enclosure 6, Item 3]

Requirement

In Mr. F. L. Clayton's letters to Mr. D. G. Eisenhut dated May 15, 1981, and June 24, 1981, concerning the above subject, Alabama Power Company committed to the following:

- Safe load paths inside containment will be defined in plant procedures.
- Alabama Power Company will revise plant procedures to minimize the time that the demineralizer hatch cover is positioned over the demineralizer during lifting operations.
- Alabama Power Company will develop procedures to cover the load handling operations for those loads listed in Table 3-1 by October 1, 1981. These procedures will meet the requirements of NUREG-0612, Section 5.1.1(2).
- Operator training will be revised to include the basic requirements of ANSI B30.2-1976 by October 1, 1981.
- Crane inspection, testing and maintenance requirements will be revised to incorporate the basic requirements of Chapter 2-2 of ANSI B 30.2-1976 by October 1, 1981.
- 6. a. Plant procedures will be reviewed with respect to the following: (1) review of procedures for installation of rigging or lifting devices and movement of load to assure that sufficient detail is provided and that instructions are clear and concise; (2) visual inspections of load bearing components of cranes, slings, and special lifting devices to identify flaws or deficiencies that could lead to failure of the component; (3) appropriate repair and replacement of defective components; and (4) verify that the crane operators have been properly trained and are familiar with specific procedures used in handling these loads, e.g., hand signals, conduct of operations, and content of procedures.
 - b. Crane operators will be trained per basic requirements of ANSI B30.2-1976 and plant procedures by October 1, 1981.
- The lifting devices identified in 2.1.3c will comply with the inspection criteria and operator qualification requirments of ANSI N14-6-1978 or ANSI 330.9-1971 as appropriate by October 1, 1981.

Response

 Safeload paths inside containment have been defined in plant maintenance procedures.

- By April 1, 1982 a precaution will be added to plant procedures to minimize the time in which a cation bed demineralizer hatch cover is suspended over its respective demineralizer cubicle.
- Alabama Power Company has completed development of load handling
 procedures meeting the requirements of NUREG-0612 Section 5.1.1(2) for those loads listed in Table 3-1.
- 4. Crane operators involved in the lifting of heavy loads at Farley Nuclear Plant have been trained in accordance with ANSI B30.2-1976. Licensed reactor operators involved in fuel manipulations and personnel involved in radwaste handling and decontamination activities are exempted from this training since they do not lift heavy loads.
- Crane inspection, testing and maintenance requirements have been revised to incorporate the basic requirements of Chapter 2-2 of ANSI B30.2-1976.
- a. Plant procedures have been reviewed and found to comply with the appropriate criteria.
 - b. Crane operators involved in the lifting of heavy loads at Farley Nuclear Plant have been trained in accordance with ANSI B30.2-1976 and appropriate plant procedures.
- The lifting devices identified in 2.1 3c comply with the inspection criteria and operator qual fication requirements of ANSI N14.6-1978 or ANSI 330.9-1971 as appropriate.

Item 11. FIRE DAMPER DEFICIENCY CORRECTION [Enclosure 6, Item 4]

Requirement

References: 1. NRC inspection of March 10 - 13, 1981 Report No. 50-348/81-06.

> Letter from Mr. F. L. Clayton, Jr. to Mr. J. P. O'Reilly, dated May 11, 1981

800.9

The reference (1) inspection reports states:

"Section 4.4.4.1.10 of the Farley Nuclear Plant Fire Protection Program Reevaluation (FPPR) dated September 15, 1977 with Amendments 1 through 4 (Amendment 4 dated January 3, 1979) states that modifications will be made to the ventilation duct penetrations of fire barriers to ochieve compliance with the NRC guidelines of Appendix A to Branch Technical Position 9.5.1, Guidelines for Fire Protection for Nuclear Power Plants Docketed Prior to July 1, 1976. Appendix A Section D.1(j) requires ventilation duct systems to be provided with "Fire door dampers" installed in accordance with National Fire Protection Association Standard No. 80 (NFPA-80), Fire Doors and Windows, at all locations where the ducts penetrate fire barriers. NFPA-80 Section 2-8,9 requires fire door assemblies to be installed in accordance with the manufacturer's installation instructions.

Contrary to the above, the fire damper was not installed in accordance with the manufacturer's installation instructions".

The reference (2) letter states:

"All other Unic 1 fire dampers will be inspected and any deficiencies corrected prior to the return to power at the next refueling outage."

Response

All Unit 1 fire dampers have been inspected. As of the present time not all of the deficiencies have been corrected. It is expected that these deficiencies will be corrected prior to return to power based upon the nature of the deficiencies, corrective responses received from engineering to date, and the time it has taken to perform the corrective action on those dampers already repaired.

At the time that Alabama Power Company committed to correcting the remaining fire damper deficiencies on Farley Unit 1 during the third refueling outage, that outage was scheduled to begin on or about March 12, 1982. Main generator problems forced Farley Unit 1 into an unexpected outage in early September, 1981. Because of the extensive repairs required to restore the generator, a decision was made to refuel the reactor during this outage in lieu of a refueling outage in the summer of 1982. Alabama Power Company is making every attempt to correct these deficiencies during the current outage. Item 12. RADIOLOGICAL EMERGENCY RESPONSE [Enclosure 6, Item 8]

Requirement

By letter dated December 29, 1979, Alabama Power Company committed to provide the following by January 1,1982:

- a. Installation date for the emergency response facility (TSC, EOF, OSC) hardware and software.
- b. Primary and backup meteorological equipment, Class A DCM and remote interrogation capability of Appendix 2 to NUREG 0654.

Response

- a. The emergency response facility hardware and software will be installed in the Technical Support Center by July 1, 1982, and is scheduled to be completed for the Emergency Operations Facility by July 1982.
- b. The primary and backup meteorological equipment has been installed and is operational.

The Farley Nuclear Plant Class A DCM System software programs have been loaded into the plant computer and are in the final testing phase. Upon completion of the final testing phase, scheduled for December 31, 1981, the Farley Nuclear Plant will have an operational Class A DCM. The Class A model's conceptual design (EDCM) is complete and has been reviewed by the NRC. The conceptual design was transmitted to the NRC by Alabama Power Letter dated February 9, 1981. The system has been designed on a modular basis. The major software components are:

- 1. Puff Processing Module
- 2. EDCM Reporting Module
- 3. System Utilities

The puff module simulates the emission of a gaseous puff from the plant vent containing a homogeneous isotopic concentration determined from standard tables in the computer files. This puff updated every 15 minutes, comprises both an elevated and a ground level component. Height of the elevated component is determined by meteorological and stream release data located in computer file tables. The puffs are tracked as they traverse a ten mile circular boundary surrounding the plant. Puffs are purged from the data base after they cross the ten mile boundary or after twelve hours from origin, whichever event occurs first.

Data gathered from meteorological conditions and effluent grab samples are filed in a computer storage table.
Item 12. RADIOLOGICAL EMERGENCY RESPONSE (Cont'd)

The EDCM reporting module utilizes the existing puff location file to simulate radiation dosage emissions based on current location of puffs, meteorological conditions, and the isotope table.

Meteorological conditions and the isotope table are located in a computer storage table. The EDCM process will be initiated by the puff process module every fifteen minutes. This will produce report parameters which include current locations, arrival times, and organ dose calculations of all puffs emitted since the start of the incident. All report parameters generated during the incident are kept in the report parameter computer file and can be viewed on demand. After an incident is terminated, a utility is also available to archive the parameters to tape. A password secured routine is also avaliable to terminate the incident. This is the only manner in which an incident is terminated and is executed from the system console.

The system has several utilities designed to insure data integrity, availability, and to edit the master file and tables. These utilities also can be utilized to generate initial files during system startup.

Alabama Power Company will provide data to the offsite groups via voice communications and/or telecopy rather than a computer data link between onsite and offsite groups. This position was described in Alabama Power Company's letter to the NRC '>ted May 19, 1981. Item 13. REACTOR COOLANT PUMP, OIL COLLECTION SYSTEM UPGRADE PER 10 CFR 50, APPENDIX R [Enclosure 6, Item 7]

Requirement

Reference: Letter from Mr. F. L. Clayton, Jr. to Messrs' S. A. Varga and A. T. Schwencer dated March 19, 1981.

Attachment 1 to the reference letter shows "anticipated outage schedule for FNP for Appendix R Planning." For FNP Unit 1, the third refueling outage is shown as 2-22-82 to 4-1-82. During that time interval, it was anticipated, as shown in attachment 2 to the reference letter, "Plan and Schedule for Fire Protection Program for Operating Power Plants Part 50.48 and 10CFR50 Appendix R" that the installation would be completed for the seismic reactor coolant pump of collection system. Such implementation in Unit 2 is to be completed by the 1st Refueling.

Response

At the present time, part of the engineering is released and at the site, it is anticipated that the balance of the engineering will be released shortly. Based upon the engineering released to date and information from engineering planning, it is anticipated that the installation of changes necessary to seismically qualify the existing PCP oil collection system will not be completed by the end of the third refueling outage. Alabama Power Company is making every effort to improve upon the current schedule for design and installation. Every effort will be made to seismically qualify the oil collection system for Unit 1 during the current refueling outage but in no case will such qualification be completed. The implementation of the Unit 2 system is still scheduled to be completed during the first refueling outage. Item 14. II.B.2, PLANT SHIELDING [Enclosures 1 and 3, item II.B.2; Enclosure 5, para 2.C.(21)(c)]

Requirement

In accordance with the Unit 2 Operating License, Alabama Power Company must, prior to April 1, 1982, complete all modifications to assure access to vital areas and protection of safety equipment following an accident resulting in a degraded core.

Response

UNIT 1

Currently work is progressing to install disconnect devices (scheduled for delivery in 'auary, 1982) during the current outage. If procurement problems occur, the installation of such devices will be completed when the unit is in cold shutdown for sufficient duration but no later than the return to power following the fourth refueling. The modification of the penetration door shielding is complete.

UNIT 2

All but two modifications to meet the above requirement have been completed. The remaining modifications entail installation of a shield door on the electrical penetration room door and installation of electrical disconnects to allow operation of eight motor operated valves (during post-accidental conditions). Current installation and procurement schedules show completion of the door modification prior to April 1, 1982. However, an outage is required to install the electrical disconnect devices that are to be located outside the electrical penctration room. When Alabama Power Company originally discussed this with the NRC during Unit 2 licensing, the intent was to have a two (2) week generator outage during February to meet the turbine generator manufacturer's requirements. This requirement will be obviated by a planned shorter seven day inspection on the turbine generator after return to power of Unit 1. This shorter outage will be for the purpose of installing Turbine Generator monitoring equipment and to perform a limited generator inspection to provide confidence of operation until the first refueling. The seven day period will be insufficient time to accomplish this change. Specifically this design change would require extension of the outage for approximately two weeks and would necessitate bringing the unit to cold shutdown which is not currently planned. No outage of sufficient duration for installation of this change is planned until the first refueling outage. Alabama Power commits to having these devices installed prior to returning to power operation following the first refueling outage or other outage of sufficient duration.

Item 15. II.K.3.5, AUTOMATIC TRIP OF REACTOR COOLANT PUMPS DURING LOSS OF-COOLANT ACCIDENT [Enclosures 1 and 3, item II.K.3.5; Enclosure 5, para 2.C.(21)(i)(2)(i)

Requirement

 By letter to the NRC dated June 30, 1981, Alabama Power makes the following commitment on this subject related to the Farley Nuclear Plant:

Within three (3) months of formal NRC notification that automatic RCP trip design modifications are required, it is Alabama Power Company's intention supply design information for Units 1 and 2.

2. The Unit 2 License item 2.C.(21)(i)(2)(i) states that with respect to tripping of RCPs, Alabama Power Company must submit to the NRC for approval either (1) an evaluation which shows that sufficient time is ailable to the operator to manually trip the RCPs in the event or a small break LOCA, or (2) a description of design modifications required to provide for an automatic pump trip. This submittal is required within three months after NRC determination of acceptability of the small break LOCA model based on comparisons with LOFT test L3-6.

Response

Westinghouse (in support of the Westinghouse Owners Group) has performed an analysis of delayed reactor coolant pump trip during small-break LOCAs. This analysis is documented in WCAP 9584 and 9585, August 1979. In addition, Westinghouse (again in support of the Westinghouse Owners Group) has performed test predictions of LOFT Experiments L3-1 and L3-6. The results of these predictions are documented in letters OG-49(3-3-81), OG-50(3-23-81) and OG-60 (6-15-81).

Based on: 1) the Westinghouse analysis, 2) the excellent prediction of the LOFT Experiment L3-6 results using the Westinghouse analytical model, and 3) Westinghouse simulator data related to operator response time, the Westinghouse and Alabama Power Company position is that automatic reactor coolant pump trip is not necessary since sufficient time is available for manual tripping of the pumps.

Our understanding of the schedule for final resolution of this issue is:

A) Once the NRC formally approves the Westinghouse model, a 3-month study period will ensue during which the Westinghouse Owners Group will attempt to demonstrate compliance with some NRC acceptance criteria for manual RCP trip. The NRC acceptance criteria will accompany their formal approval of the Westinghouse models.

- Item 15. II.K.3.5, AUTOMATIC TRIP OF REACTOR COOLANT PUMPS DURING LOSS OF-COOLANT ACCIDENT (Cont'd)
 - B) If, at the end of the 3-month period, the Westinghouse Owners Group cannot show compliance with the acceptance cirteria, the NRC will formally notify utilities that they must submit an automatic RCP trip design.