Public Service Company of Colorado

16805 Road 19 1/2, Platteville, Colorado 80651-9298

December 17, 1981
Fort St. Vrain
Unit No. 1
P-81318

Mr. John T. Collins, Regional Administrator
Region IV
Nuclear Regulatory Commission
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76011

Reference: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Collins:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/81-072, Preliminary, submitted per the requirements of Technical Specification AC 7.5.2(b)1.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/81-072.

Very truly yours,

Don Warembourg by FSBorst

Don Warembourg
Manager, Nuclear Production

DW/cls

Enclosure

cc: Director, MIPC

IE22
S
1/1

OCCURRENCE REPORT DISTRIBUTION

	Number of Copies	
Department of Energy San Francisco Operations Office Attn: California Patent Group 1333 Broadway Oakland, California 94612	1	(P Letter)
Department of Energy Mr. Glen A. Newby, Chief HTR Branch Division of Nuclear Power Development Mail Station B-107 Washington, D.C. 20545	1	(P Letter)
Department of Energy Attn: Project Manager 110 West A Street, Suite 460 San Diego, California 92101	1	(P Letter)
~~Mr. Karl V. Seyfrit, Director~~ J. T. Collins Region IV ~~Office of Inspection and Enforcement~~ Nuclear Regulatory Commission 611 Ryan Plaza Drive Suite 1000 Arlington, Texas ~~76012~~ 76011	1	(Original of P Letter and Copy of LER)
Director Office of Management Information and Program Control U. S. Nuclear Regulatory Commission Washington, D.C. 20555	1	(P Letter, LER)
Mr. Richard Phelps FSV, GA, Site Representative General Atomic Company P. O. Box 426 Platteville, Colorado 80651	10	(Original of FPLG Letter plus Two Copies, One Copy of P Letter, One Copy of LER)
Mr. Bill Lavalee Nuclear Safety Analysis Center 3412 Hillview Avenue P. O. Box 10412 Palo Alto, California 94303	1	(P Letter, LER)
NRC Resident Site Inspector	1	(P Letter, LER)

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651-9298

REPORT NO. 50-267/81-072/03-L-0

Preliminary

IDENTIFICATION OF OCCURRENCE:

During a routine surveillance check performance on November 17, 1981, the setpoint of the PCRV rupture disk assembly M-11701 was found to be 847 psig. Since this setpoint is higher than the 832 psig ± 1% allowable by Fort St. Vrain Technical Specification LSSS 3.3, Table 3.3.1.2(c), it is being reported per Fort St. Vrain Technical Specification AC 7.5.2(b)1.

EVENT DESCRIPTION:

With the reactor shutdown and depressurized for scheduled maintenance, the high set rupture disk M-11701 was removed from service for a scheduled surveillance calibration bench test. The "as found" pressure setpoint was 847 psig. The acceptance criterion of LSSS 3.3, Table 3.3.1.2(c) requires the set pressure to be 832 psig ± 1% (824 psig to 840 psig).

Refer to Figure 1. Before the test gas pressure is applied, the cutter, (1), is removed, and a special ring is installed to limit the travel of the Belleville washers (2). The washers snap-over when the set pressure is reached. Adjustment of the set pressure is made by positioning of the threaded ring (3). When the set pressure is reached, the diaphragm and washers travel a few thousandths of an inch, which produces an audible "thump." The plant instrumentation personnel know from previous experience that the setpoint of the Belleville washers decreases slightly with subsequent snap-overs during testing. Therefore, the snap-over test was conducted two additional times with snap-overs occurring at 831 psig and 831 psig, respectively. The last two tests were within the specified acceptable range. The average of the three tests was 836 psig, which is satisfactory, so no further adjustments were required. The remainder of the surveillance was completed, and the rupture disk assembly was returned to service.

During this surveillance interval, the low set rupture disk, M-11702, was operable and provided the required overpressure protection. M-11702 was last calibrated in 1980, and was left within the specified setpoint range.

CAUSE
DESCRIPTION:

The rupture disk movement setpoint being out of tolerance was caused by drift of the Belleville washer operating point over the period since its' last calibration surveillance.

CORRECTIVE
ACTION:

The snap-over test was conducted three times with the last two of the tests within the required setpoint band. The remainder of the surveillance was completed, and the rupture disk assembly was returned to service.

An evaluation of the rupture disks is being conducted by the Public Service Company of Colorado's Nuclear Engineering Division. The results of this evaluation will be included in a future supplemental report.

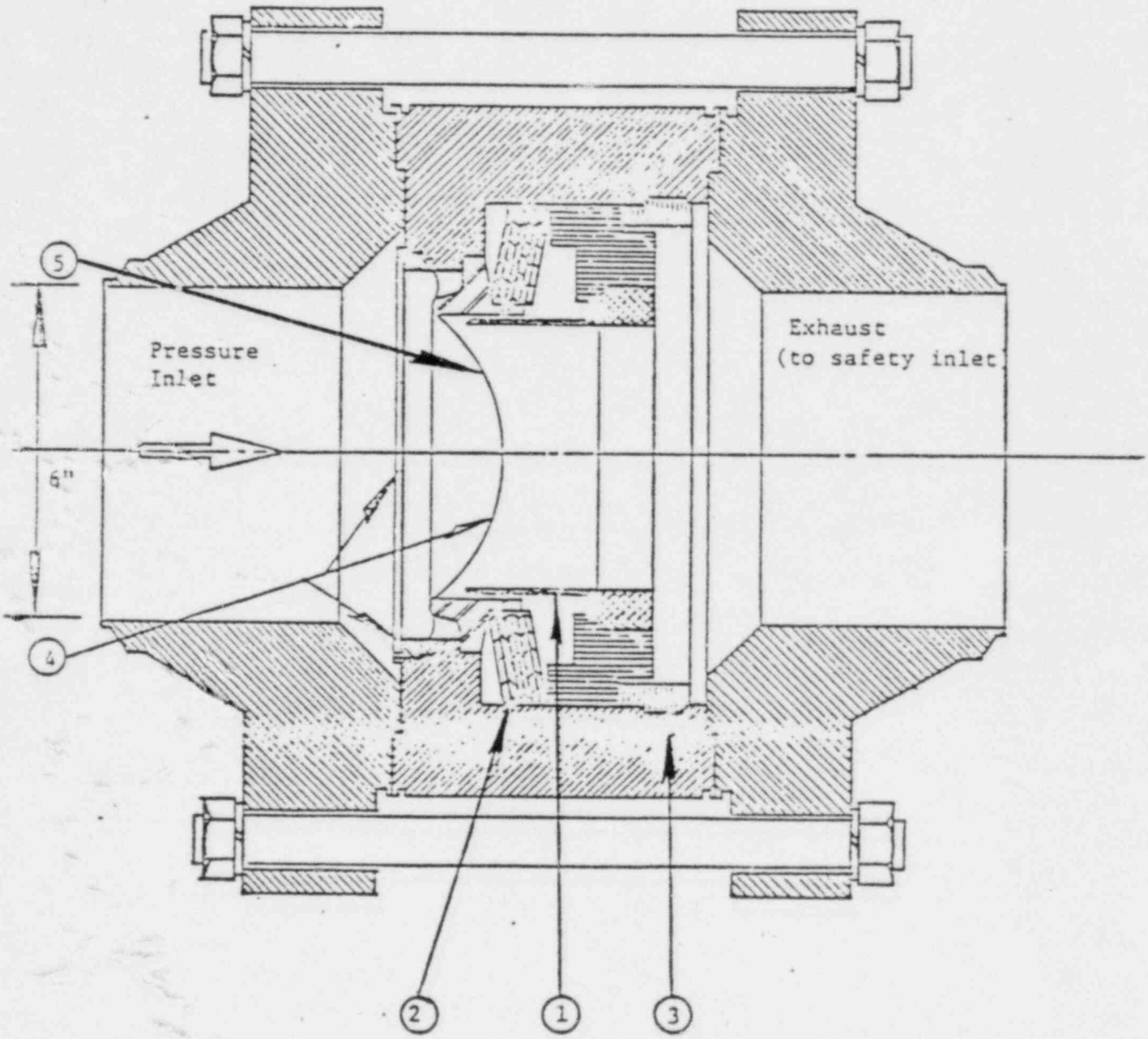

FIGURE 1

Prepared By: Robert A. Dickerson
Robert A. Dickerson
Technical Services Technician

Reviewed By: Charles Fuller
Charles Fuller
Technical Services Engineering Supervisor

Reviewed By: Edwin D. Hill Martin Block
Edwin D. Hill
Station Manager

Approved By: Don Warembourg by F.J. Borst
Don Warembourg
Manager, Nuclear Production