NRC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) Ø CONT REPORT 0 10 15 1 10 10 SOURCE DOCKET NUMBER EVENT DATE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10 (NP-33-81-89) On 11/16/81 the NSSS vendor, Babcock and Wilcox, confirmed that a new analysis showed a general increase in the Reactor Protection System (RPS) instrument string errors. This reduced the margin between the actual RPS trip setpoints in the -4 station and the trip points in the Tech Specs. This finding is reportable under T.S. 6.9.1.9. There was no danger to the public or station personnel. The Cycle 2 Tech 6 Spec RPS trip setpoints still contained identifiable safety margins to offset the increased instrument error such that the functional requirements of the system would have been met. SYSTEM COMP CODE COMPONENT CODE SUBCODE SUBCODE I A GI 12 12 1Z Z Z (15 21 (16) SEQUENTIAL REPORT NO. OCCURRENCE REPORT REVISION LER RO TYPE Nori. 01 71 NUMBER Ø METHOD FUTURE SEFECT ON PLANT ATTACHMENT SUBMITTED NPRD-4 PRIME COMP. COMPONENT FORM SUR 217 10 10 10 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) This finding is due to additional testing on the RPS instruments which showed increased The actual RPS trip setpoints in the plant have been adjusted under Facility errors. Change Request 81-295 to account for the increased instrument errors. The vendor has peen authorized to perform an analysis to determine the new Technical Specification RPS trip setpoints for the upcoming Cycle 3 Technical Specification revisions. METHOD OF DISCOVERY % POWER OTHER STATUS DISCOVERY DESCRIPTION (32) D (31) Engineering analysis by Babcock & Wilcox CONTENT ACTIVITY RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE (36 (34) PERSONNEL EXPOSURES SCHETION (39) PERSONNEL INJUNICS DESCRIPTION (41) NUMBER 01 Ø (40) NA OSS OF OH DAMAGE TO FACILITY (43 DESCRIPTION NA (42) 8112290404 811216 PDR ADDCK 05000346 PUBLICITY DESCRIPTION (45 (44) NA PHONE (419) 259-5360 VR 81-194 Frank Chen/Jan Stotz NAME OF PREPARER.

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-81-89

DATE OF EVENT: November 16, 1981

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: An engineering analysis indicated a reduction in the safety margin of some Reactor Protection System (RPS) trip setpoints.

Conditions Prior to Occurrence: The unit was in Mode 5 with Power (MWT) = 0 and Load (Gross MWE) = 0.

Description of Occurrence: On November 16, 1981, the NSSS vendor (Babcock and Wilcox) confirmed that a new analysis showed various increases in the RPS instrument string errors. This reduced the margin between the actual RPS trip setpoints in the station and the trip points in Technical Specifications. However, there is still sufficient margin to maintain functionality of the RPS. This finding is reportable under Technical Specification 6.9.1.9 as RPS setpoints which are found to be less conservative than those established by Technical Specifications but which do not prevent the fulfillment of the functional requirement of the affected system.

Designation of Apparent Cause of Occurrence: This finding is due to additional testing done by IEEE-323 1971 methods on RPS instruments. This testing revealed increases in module errors beyond the originally calculated values.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. The Cycle 2 Technical Specification RPS trip setpoints still contain identifiable safety margins to offset the increased instrument error such that the functional requirements of the system would have been met.

Corrective Action: As a precautionary measure, the actual RPS trip setpoints in the plant have been more stringently adjusted under Facility Change Request 81-295 to account for the increased instrument errors. The adjustments were completed on November 22, 1981.

The NSSS vendor has been authorized to perform a detailed analysis to determine the new Technical Specification RPS trip setpoints and allowable limits for the upcoming Cycle 3 Technical Specification revisions.

Failure Data: There have been no reported similar occurrences at Davis-Besse, however this finding has been previously reported to the NRC by other plants.

LER #81-074