

OPERATING DATA REPORT

DOCKET NO. 050-0331
 DATE December 14, 1981
 COMPLETED BY J. Van Sickle
 TELEPHONE 319-851-5611

OPERATING STATUS

- 1 Unit Name: Duane Arnold Energy Center
2. Reporting Period: November, 1981
3. Licensed Thermal Power (MWt): 1658
- * 4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:
-
9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____
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Notes

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	8,016	59,880
12. Number Of Hours Reactor Was Critical	720	5,581.5	42,750.2
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	720	5,400.6	41,677.6
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	691,361	6,501,630	52,500,268
17. Gross Electrical Energy Generated (MWH)	228,252	2,191,976	17,594,663
18. Net Electrical Energy Generated (MWH)	211,886	2,051,618	16,463,656
19. Unit Service Factor	100.0%	67.4%	69.6%
20. Unit Availability Factor	100.0%	67.4%	69.6%
21. Unit Capacity Factor (Using MDC Net)	57.1%	49.7%	53.4%
22. Unit Capacity Factor (Using DER Net)	54.7%	47.6%	51.1%
23. Unit Forced Outage Rate	0.0%	4.0%	17.3%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

* Turbine Rating: 565.7 MWe
 Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

AVERAGE DAILY UNIT POWER LEVEL

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 Duane Arnold
 UNIT Energy Center
 DATE December 14, 1981
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MONTH November, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>219</u>	17	<u>331</u>
2	<u>235</u>	18	<u>326</u>
3	<u>262</u>	19	<u>322</u>
4	<u>250</u>	20	<u>296</u>
5	<u>334</u>	21	<u>290</u>
6	<u>316</u>	22	<u>275</u>
7	<u>321</u>	23	<u>309</u>
8	<u>323</u>	24	<u>279</u>
9	<u>329</u>	25	<u>264</u>
10	<u>324</u>	26	<u>252</u>
11	<u>327</u>	27	<u>254</u>
12	<u>325</u>	28	<u>253</u>
13	<u>323</u>	29	<u>255</u>
14	<u>326</u>	30	<u>255</u>
15	<u>329</u>	31	<u>255</u>
16	<u>327</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1981

DOCKET NO. 050-0331
 UNIT NAME Duane Arnold Energy Ctr.
 DATE December 14, 1981
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 TELEPHONE 319-851-5611

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None									

1 F: Forced
S: Scheduled

2 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit I - Same Source

REFUELING INFORMATION

Docket No. 050-0331
Unit Duane Arnold Energy C
Date December 14, 1981
Completed by J. Van Sicken
Telephone 319-851-5671

1. Name of facility.
A. Duane Arnold Energy Center
2. Scheduled date for next refueling shutdown.
A. Fall, 1982
3. Scheduled date for restart following refueling.
A. Unknown
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
A. Yes. New MAPI:HGR tables will have to be included in Technical Specifications.
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
A. Unknown at this time.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
A. a) 368 b) 448
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
A. 2050
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
A. 1998

MAJOR SAFETY RELATED MAINTENANCE

Docket No. 050-0331
Unit Duane Arnold Energy Center
Date December 14, 1981
Completed by J. Van Sickle
Telephone 319-851-5611

DATE	SYSTEM	COMPONENT	DESCRIPTION
11-2-81	CRD Hydraulic	V-17-111	Replaced valve
11-4-81	Containment Atmospheric Control	SV-8116B	Replaced coil
11-6-81	River Water Supply	CV-4909	Replaced SV-4909
11-6-81	Containment Atmospheric Control	SV-8103B	Replaced coil
11-6-81	Containment Atmospheric Control	AN 8181A	Replaced flow controller
11-21-81	Containment Atmospheric Control	AN 8181A	Installed new chemicals

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 11-1 At the beginning of the report period the plant was operating at 237 MWe. The plant was being operated at reduced power levels due to lack of system demand.
- 11-4 Control rod group 36 was withdrawn to position 48. A power increase was begun.
- 11-5 The plant was operating at 341 MWe.
- 11-6 During normal operation while performing an operational inspection, a leak was discovered in the reactor water cleanup system outboard of both isolation valves.
- RO Report 81-041
- 11-14 During a weekly control rod exercise surveillance test, an overtravel alarm was received for control rod 26-15, indicating the CRD had uncoupled.
- RO Report 81-042
- 11-17 During power operation, the instrument line drain valve, V-18-20, for "B" scram instrument volume level switch LS-1861 was found locked open.
- RO Report 81-039
- 11-18 During normal operation, it was found that reactor water cleanup system return isolation valve MOV 2740 would not cycle.
- RO Report 81-040
- 11-24 During normal operation secondary containment isolation damper 1V-AD-15B began cycling and was removed from service.
- RO Report Pending
- 11-24 While testing the "A" control building standby filter unit, isolation damper 1V-AD-31A was found inoperable.
- RO Report Pending

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 11-27 1V-AD-31A was tested and declared operable.
- 11-28 The "A" reactor feed pump and "A" condensate pump were secured.
- 11-30 The plant was operating at 273 MWe.