

~~ATTACHMENT 16~~

REQUEST FOR ADDITIONAL INFORMATION

EQUIPMENT ENVIRONMENTAL QUALIFICATION (EEQ)
REVIEW OF LICENSEES' RESOLUTION OF OUTSTANDING ISSUES
FROM NRC EQUIPMENT ENVIRONMENTAL QUALIFICATION SAFETY
EVALUATION REPORTS (SER) AND TMI ACTION PLAN INSTALLED EQUIPMENT

Northern States Power Company
Prairie Island Unit # 2

NRC Docket No. 50-282, -306

December 9, 1981

NRC TAC No's 42500, 42501

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BACKGROUND

Franklin Research Center (FRC) of Philadelphia, Pa. is providing assistance to the U.S. Nuclear Regulatory Commission (NRC) for the equipment environmental qualification (EEQ) review of operating reactors. FRC will perform an EEQ review of the Licensee's 90-day response to outstanding issues from the NRC Equipment Environmental Qualification Safety Evaluation Report (SER). ~~_____~~. The review will be limited to safety-related equipment potentially exposed to a harsh environment. The results will be presented in the form of a technical evaluation report for each plant.

This request for additional information (RPI) is the result of an evaluation of the information provided by letters dated October 31, 1980 [1] and August 26, 1981 [2].*

A. FRC REVIEW OF THE LICENSEE'S 90-DAY RESPONSE TO THE NRC EEQ SER

Information RequestedDate Received by FRC**

1. FRC requests the LOCA/HELB profiles, for inside and outside containment, described on page 4 of Reference 2.
2. In reference to the Licensee's 90-day response [2] to the NRC SER [3], a legible single copy of each of the following qualification documents is requested in order that the FRC evaluation may proceed:
 - a. Okonite Company Letter to A. Smith (NSP)
August 31, 1978
(A.24.6)***
 - b. Kerite Company Report KPT-LVC-1
April 13, 1977 (proprietary)
(A.24.8)

*Numbers in brackets refer to citations found in the list of references.

**This column will be completed by FRC as the requested information is received.

***Numbers in parentheses refer to the Licensee's SCEW sheet numbers.

Date Received by FRC**

- c. Barton Test Report No. RJ-288A-1
October 1979
(A.21.1)
- d. GE, "Insulating Materials Product Data
74010A Epoxy Resin and 74010 Epoxy
Catalyst," March 24, 1964, and "Effect of
Radiation on Materials"
(A.24.4)
- e. ACME-Cleveland Development Co.
Qualification of NAMCO Controls Limit
Switch - Model EA-170, Rev. 1
July 24, 1978
(A.6.5, A.6.9, A.17.2)
- f. ACME-Cleveland Development Co. Test Plan
August 31, 1977
(A.16.6, A.12.6, A.12.7, A.2.3, A.2.4,
A.4.4, A.6.3, A.6.4, 6.6, 14.3, A.15.1,
A.15.2, A.16.8, A.19.5, A.19.6)
- g. "Comparative Heat Resistance of Hypalon
and Neoprene," by K. Whitlock
Elastomers Research Division of DuPont
(A.24.9)
- h. EDS Calculation File Nos. 0910-204EQ-01
0910-205-Okonite-01
0910-205-Kerite
0910 205-BIW-01
- i. EDS Report Nos. 04-0910-13, June 1981
04-0910-07, June 1981
- j. J. M. Allen (Mobil Oil)
Letter to J. Sorenson (NSP)
February 7, 1980
(A.24.2)
- k. D. R. Jones (Chevron USA, Inc.)
Correspondence with A. Smith (NSP)
October 30, 1979
(A.24.3)
- l. R. E. Shoults (Westinghouse)
Letter to G. A. Reed (Wis-Minn Power)
January 19, 1977
(A.24.3)

Date Received by FRC**

- m. H. C. Lauroesch (CE)
Correspondence with J. King (GE)
August 7, 1978
(A.24.4)
- n. GE Letter to A. Smith (NSP)
November 21, 1978 (Attachment)
(A.24.4)
- o. Robert Kantner (NAMCO)
Letter to Paul T. Roska (NSP)
November 18, 1980
(A.6.5, A.6.9)
- p. Nuclear Radiation and Switch Applications
October 7, 1974
(A.6.7)
- q. Honeywell Catalog 50, p. E-2
(A.6.7)
- r. B. Bondow (E-M)
Letter to J. Sorenson (NSP)
March 25, 1980
(A.7.1)
- s. NSC Letter to NSP
Component Aging Evaluation
September 29, 1980
(A.7.1, A.24.1)
- t. Okonite Company Test Procedure sent to
Mr. Albrecht (NSP) on April 20, 1978
(A.24.6)

REFERENCES

1. D. E. Gilberts, L. O. Mayer
Letter to J. G. Keppler, NRC.
Subject: Final Response to IE Bulletin 79-01B, Prairie
Island Nuclear Generating Plant
Northern States Power Company, 31-Oct-80
2. L. O. Mayer
Letter to NRC. Subject: Prairie Island Nuclear
Generating Plant - Response to IE Bulletin 79-01B
Safety Evaluation Report
Northern States Power Company, 26-Aug-81
3. Office of Nuclear Reactor Regulation
Safety Evaluation Report for Prairie Island Units 1 and 2
Environmental Qualification of Safety-Related
Electrical Equipment
NRC, 12-May-81

