



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

December 9, 1981

NUCLEAR PRODUCTION DEPARTMENT

U. S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:



SUBJECT: Grand Gulf Nuclear Station  
Units 1 and 2  
Docket Nos. 50-416 and 50-417  
File 0260/0862/L-340.0  
Transmittal of Proposed FSAR  
Change and Response to NRC  
Informal Question  
AECM-81/479

In response to your request for additional information, Mississippi Power & Light Company (MP&L) is submitting the enclosed information.

This information represents a proposed change to the Grand Gulf Nuclear Station Final Safety Analysis Report (FSAR). This proposed FSAR change will be incorporated into the next available amendment to the FSAR. If you have any questions or require further information, please contact this office.

Yours truly,

L. F. Dale  
Manager of Nuclear Services

JHS/JGC/JDR:ph  
Attachment

cc: Mr. N. L. Stampley (w/a)  
Mr. R. B. McGehee (w/a)  
Mr. T. B. Conner (w/a)  
Mr. G. B. Taylor (w/a)

Mr. Richard C. DeYoung, Director (w/a)  
Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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BRANCH: Quality Assurance Branch

CONCERN: The response to 260.1 [b(3)] concerning reactor internals, transmitted by AECM-81/345, dated September 3, 1981, requires clarification or additional information.

RESPONSE: The following is a revised response to Item b-3 identified in Question 260.1:

b-3 Item I.6 of FSAR Table 3.2-1 includes those non-safety class internal structures such as feedwater spargers, steam dryers, shroud head and steam separator assembly, in-core guide tubes and stabilizers, and surveillance sample holders. These structures do not perform a safety function and are not required to prevent or mitigate the consequences of accidents. A failure of the feedwater sparger will not prevent transmission of cooling water to the core or affect the safety of the reactor system. Although, these structures are not safety-related, they are so designed that they will not adversely affect the safety function of the safety related structures. The appropriate portions of the MP&L Operational Quality Assurance Program will be applied to these structures.

Appendix B of the Q-List and FSAR Table 3.2-1 will be revised to reflect this response.