LICENSEE EVENT REPORT

	CONTROL BLOCK: [] [] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1 8	A L B R F 3 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5 6 6 6 6 6 6 6 6 6 6 6 6 6 6 6 6 6 6
CON'T	REPORT 1 6 0 5 0 0 0 2 9 6 7 0 6 1 6 8 1 8 1 1 2 3 8 1 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
0 2	During normal operation, the fire protection preaction sprinkler systems for
	reactor building elevations 565, 593, and 621 were found isolated.
0 3	(See T.S. 3.11.A.1.a) There was no danger to the health or safety of the public.
0 4	
0 5	There were no previous similar events. There are no redundant systems.
0 6	
0 7	
0 8	9 SYSTEM CAUSE CAUSE COMP. VALVE
0 9	CODE SUBCODE COMPONENT CODE SUBCODE SU
	17) REPORT NUMBER 8 1 1 21 22 23 24 26 27 28 29 30 31 32
	ACTION FUTURE COMPONENT TAKEN ACTION ON PLANT ON
_	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) [Lack of familiarity with the isolution valve operating characteristics caused]
1 0	
1 1	personnel to believe valve was open. The Mueller type AWWA valve was opened and the
12	systems were returned to normal. The operator involved has been instructed in the
1 3	isolation valve characteristics. The event will also be covered in operator supple-
	mental training by 12/1/81.
1.5	FACILITY SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION (32) E 28 0 9 8 29 NA C 31 Inspector observed.
	ACTIVITY CONTENT
1 6	Z 33 Z 34 NA
117	NUMBER DESCRIPTION (39) NA NA
7 8	PERSONNEL INJURIES NUMBER DESURIPTION 41
	NA NA
(TE	EGSS OF OH DAMAGE TO FACILITY (43) TYPE DESCRIPTION NA
	90 8111240561 811123 NRC USE ONLY
2 0	PDR ADOCK 05000296 PDR NA LILILIAN 45 PDR NA BOOK 05000296
	NAME OF PREPARER Gene Holder PHONE (205) 729-6134

LER SUPPLEMENTAL INFORMATION
BFRO-50- 296 / 81029 R1 Technical Specification Involved 3.11.A.1.a
Reported Under Technical Specification 6.7.2.a(2) *Date due NRC: N/A
Date of Occurrence 6/16/81 Time of Occurrence 1730 Unit 3
Identification and Description of Occurrence: The fire protection preaction sprinkler systems for reactor building elevations 565 593, and 621 were found isolated. This system had been taken out of service the previous day, and a patroling fire watch established per T.S. 3.11.A.3. The hold order, under which this maintenance was performed, was lifted. The valve was manual opened, subsequently, the system was checked on the three elevations in the reactor Conditions Prior to Occurrence: Unit 1 refueling outage (See attachment 2)
Unit 2 modification outage
Unit 3 at 98%
Established patroling fire watch.
Apparent Cause of Occurrence:
Lack of familiarity with the isolation valve operating characteristics.
Lack of familiarity with the isolation value spaces
Analysis of Occurrence: There was no danger to the health or safety of the public, no release of activity, no damage to the plant or equipment, and no resulting significant chain of events.
Corrective Action:
The valves were opened, returning the systems to normal, and the operator involved has been instructed in the isolation valve operating characteristics. The event will also be covered in operator supplemental training by December 1, 1981.

Failure Data:

*Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision:

ATTACHMENT 2

LER SUPPLEMENTAL INFORMATION BFRO-50-296/81029 R1

Identification and Description of Occurrence (Continued)

building. He then notified the shift engineer that the system was in service and relieved the fire watch. It appears that the valves were not opened (there is no position indication on the valves) and that the observed pressure was the result of the system maintaining pressure due to the isolated condition. The pressure on the isolated portion subsequently bled down over a period of time and was noted to be low by an NRC inspector. At 1100 on the day the system pressure was noted low, the isolation valves were rechecked and found to be not fully opened. The isolation valves were fully opened and system pressure reestablished.