



**Florida
Power**
CORPORATION

November 17, 1981

File: 3-0-1-a
#3F-1181-23

Mr. John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Rupture Matrix Signals

Dear Mr. Stolz:

Enclosed are forty (40) copies of Revision 1 to the report "Evaluation of Steam Line Break Consequences Associated With Removal of Rupture Matrix Signals From Emergency Feedwater Valves".

Florida Power Corporation submitted the original report on June 2, 1980. The Nuclear Regulatory Commission performed an independent analysis and the results did not agree with this report. Further review of the original analysis revealed an incorrect assumption regarding the mass release rate (exit steam flow rate was assumed rather than the appropriate steam flow rate associated with cold feedwater at 380 gpm). The results of the later analysis were discussed in a telephone conversation and a meeting on July 11, 1980. A Florida Power Corporation letter dated July 16, 1980, documented that the containment design pressure and temperature limits would be reached in 65 minutes and 36 minutes, respectively. The July 16 letter also committed to perform an additional analysis to determine if the reactor would return to critical sooner than 36 minutes. This report "Evaluation of Reactivity Response For a Steam Line Break Event with Unterminated Emergency Feedwater Flow" was submitted September 3, 1981. This revision to the original report completes all documentation to justify the removal of the rupture matrix signals.

Sincerely,

William A. Cross

William A. Cross
Manager, Nuclear Licensing

Klein(11/16)C3-3

8111240423 811117
PDR ADOCK 05000302
P PDR



*A001
3
1/1*