

Mr. Boyce M. Grier, Director
Office of Inspection & Enforcement, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

No. 2-81-07/3L

Dear Mr. Grier:

This LER concerns failure to adjust average power range monitor (APRM) gains during control rod and recirculation flow adjustments. Applicable Technical Specifications are 2.1.A and 2.1.A Bases.

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 PA P B S 2 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 _____ 5

CON'T
01 REPORT SOURCE L 6 0 5 0 - 0 2 7 7 7 0 1 0 3 8 1 8 0 2 0 2 8 1 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 During control rod and recirc flow adjustments with the unit at power,

03 the APRM flux scram trip setting was between 2.88 and 5.60% higher than

04 the setting specified in Tech. Spec. 2.1.A.1. It was out of adjustment

05 for approximately 6 hours and was promptly reset when discovered. Based

06 on indicated core maximum fraction of critical power during this period,

07 and as discussed in Tech. Spec. Bases 2.1.A, no safety limit would

08 have been exceeded had a worst case transient occurred.

09 I A 11 A 12 X 13 Z Z Z Z Z Z Z 14 Z 15 Z 16

17 LER NO. REPORT NUMBER 8 1 21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPR-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

0 E 18 Z 19 Z 20 Z 21 0 0 0 0 22 N 23 N 24 Z 25 Z 26 Z 27 Z 28 Z 29 Z 30 Z 31 Z 32 Z 33 Z 34 Z 35 Z 36 Z 37 Z 38 Z 39 Z 40 Z 41 Z 42 Z 43 Z 44 Z 45 Z 46 Z 47 Z 48 Z 49 Z 50 Z 51 Z 52 Z 53 Z 54 Z 55 Z 56 Z 57 Z 58 Z 59 Z 60 Z 61 Z 62 Z 63 Z 64 Z 65 Z 66 Z 67 Z 68 Z 69 Z 70 Z 71 Z 72 Z 73 Z 74 Z 75 Z 76 Z 77 Z 78 Z 79 Z 80 Z 81 Z 82 Z 83 Z 84 Z 85 Z 86 Z 87 Z 88 Z 89 Z 90 Z 91 Z 92 Z 93 Z 94 Z 95 Z 96 Z 97 Z 98 Z 99 Z 100

CAUSE DESCRIPTION AND CONNECTIVE ACTIONS 27

10 The event was caused by an engineer's failure to make necessary adjust-

11 ments to the APRM gains. The gains were promptly reset when the error

12 was discovered. A discussion was held by the station Reactor Engineer

13 with appropriate nuclear engineering personnel concerning the importance

14 of monitoring and timely adjustment of APRM gains.

15 FACILITY STATUS 20 % POWER 21 OTHER STATUS 30 METHOD OF DISCOVERY 32

0 0 0 5 4 21 N/A A 11 Reactor Engr. Observation

16 ACTIVITY CONTENT 10 RELEASED OF RELEASE 11 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

0 Z 13 Z 14 N/A N/A

17 PERSONNEL EXPOSURES 18 NUMBER 19 TYPE 20 DESCRIPTION 39

0 0 0 0 17 Z 18 N/A

18 PERSONNEL INJURIES 19 NUMBER 20 DESCRIPTION 41

0 0 0 0 40 N/A

19 LOSS OF OR DAMAGE TO FACILITY 20 TYPE 21 DESCRIPTION 43

0 Z 42 N/A

20 PUBLICITY 21 ISSUED 22 DESCRIPTION 45

0 N 44 N/A

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