



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

May 6, 1993

Docket No. 50-321

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: REQUEST FOR RELIEF FROM ASME SECTION XI TO DEFER A POST-REPAIR
HYDROSTATIC TEST ON A CLASS 1 INSTRUMENT LINE NOZZLE SAFE-END -
EDWIN I. HATCH NUCLEAR PLANT, UNIT 1 (TAC M86179)

By letter dated April 9, 1993, you reported the discovery of a through-wall leak in the Unit 1 reactor pressure vessel (RPV) instrument line N11B nozzle safe-end. The leak was located in the base metal adjacent to the fillet weld attaching a coupling to the safe-end. The observed leakage rate was approximately one drop per minute under static water pressure with the refueling cavity flooded. Four similar instrument nozzles were inspected with acceptable results.

The instrument line is a Class 1 component. A post-repair hydrostatic test is required, prior to returning the line to service, in accordance with the American Society of Mechanical Engineers (ASME) Section XI Code, 1980 Edition with addenda through winter 1981. In order to perform the above test, it would be necessary to include the RPV and all Class 1 piping and components that cannot be isolated from the RPV. You stated that a Code repair has been implemented and that compliance with Code requirements is not practical. You also proposed an alternative to performing the Code required test prior to plant startup. The alternative test is to perform liquid penetrant non-destructive examinations and a VT-2 visual examination on the replacement welds. You also stated that you plan to perform a post-repair hydrostatic test at the end of Hatch Unit 1 inservice inspection interval. Under the current schedule, the subject test would be performed during the spring 1996 refueling/maintenance outage.

The enclosed Safety Evaluation provides the results of the NRC staff's review of your relief request. The staff finds that the proposed alternative examination is acceptable in that it should ensure the structural integrity of the piping and components.

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May 6, 1993

Therefore, pursuant to 10 CFR 50.55a(a)(3)(i), the staff concludes that the proposed alternative to the requirements of ASME Code provides an acceptable level of quality and safety and may be authorized until spring 1996 refueling/maintenance outage when the Code required testing will be performed.

Sincerely,

/s/

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
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Mr. W. G. Hairston, III

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May 6, 1993

Therefore, pursuant to 10 CFR 50.55a(a)(3)(i), the staff concludes that the proposed alternative to the requirements of ASME Code provides an acceptable level of quality and safety and may be authorized until spring 1996 refueling/maintenance outage when the Code required testing will be performed.

Sincerely,

David B. Matthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
See next page

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