Docket No. 50-423 B14446

# Attachment 1

Millstone Nuclear Power Station, Unit No. 3

Proposed Revision to Technical Specifications Containment Leakage Marked Up Pages of Technical Specifications

April 1993

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#### CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

- 3.6.1.2 Containment leakage rates shall be limited to:
  - a. An overall integrated leakage rate of less than or equal to L 0.65% by weight of the containment air per 24 hours at Pa, 53.27 psia (38.57 psig);

January 25, 1997

- A combined leakage rate of less than 0.60 L, for all penetrations b. . and valves subject to Type B and C tests, when pressurized to Pa; and
- A combined leakage rate of less than or equal to 0.042 L, for all C . penetrations identified in Table 3.6-1 as Enclosure Building bypass leakage paths when pressurized to P\_.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the measured overall integrated containment leakage rate exceeding 0.75 L, or the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 L, or the combined bypass leakage rate exceeding 0.042 L, restore the overall integrated leakage rate to less than 0.75 L, the combined leakage rate for all penetrations subject to Type B and C tests to less than 0.60 L , and the combined bypass leakage rate to less than 0.042 L prior to increasing the Reactor Coolant System temperature above 200°F.

## SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50 using methods and provisions of ANSI N45.4-1972 (Total Time Method) and/or ANSI/ANS 56.8-1981 (Mass Point Method):

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals#during shutdown at a pressure not less than P, 53.27 psia (38.57 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection;
- If any periodic Type A test fails to meet 0.75 L, the test schedule for subsequent Type A tests shall be reviewed and approved by the b. Commission. If two consecutive Type A tests fail to meet 0.75 L, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet 0.75 L, at which time the above test schedule may be resumed;

\* The test interval for conducting a Type A test shall be extended to allow the second Type A test, within the first ten-year service prival, to be conducted during the cycle 4 refueling outage. This extension expires upon completion of the cycle 4 refueling outage. MILLSTONE - UNIT 3 24 5 2 MILLSTONE - UNIT 3 3/4 6-2

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# Attachment 2

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Millstone Nuclear Power Station, Unit No. 3

Proposed Revision to Technical Specifications Containment Leakage Retyped Pages of Technical Specifications

April 1993

#### CONTAINMENT SYSTEMS

#### CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- An overall integrated leakage rate of less than or equal to L, 0.65% by weight of the containment air per 24 hours at P<sup>a</sup>, 53.27 psia (39.4 psig);
- A combined leakage rate of less than 0.60 L<sub>a</sub> for all penetrations and valves subject to Type B and C tests, when pressurized to P<sub>a</sub>; and
- c. A combined leakage rate of less than or equal to 0.042 L for all penetrations identified in Table 3.6-1 as Enclosure Building bypass leakage paths when pressurized to P.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the measured overall integrated containment leakage rate exceeding 0.75 L, or the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 L, or the combined bypass leakage rate exceeding 0.042 L, restore the overall integrated leakage rate to less than 0.75 L, the combined leakage rate for all penetrations subject to Type B and C tests to less than 0.60 L, and the combined bypass leakage rate to less than 0.042 L prior to increasing the Reactor Coolant System temperature above 200°F.

## SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50 using methods and provisions of ANSI N45.4-1972 (Total Time Method) and/or ANSI/ANS 56.8-1981 (Mass Point Method):

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40  $\pm$  10 month intervals\* during shutdown at a pressure not less than P, 53.27 psia (38.57 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection;
- \* The test interval for conducting a Type A test shall be extended to allow the second Type A test, within the first ten-year service period, to be conducted during the Cycle 4 refueling outage. This extension expires upon completion of the Cycle 4 refueling outage.

## SURVEILLANCE REQUIREMENTS (Continued)

b. If any periodic Type A test fails to meet 0.75 L, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet 0.75 L, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet 0.75 L at which time the above test schedule may be resumed;