Duquesne Light Company Beaver Valley Power Station

Beaver Valley Power Station P.C. Box 4 Shippingport, PA 15077-0004

JOHN D. SciCBER Senior Vice President and Chief Nuclear Officer Nuclear Power Division

April 19, 1993

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U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Subject: Beaver Valley Power Station, Unit No. 1 and No. 2 BV-1 Docket No. 50-334, License No. DPR-66 BV-2 Docket No. 50-412, License No. NPF-73 Additional Information Supporting TSCR 208/74 (TAC Nos. M85819/M85820)

Attached are summaries of the Beaver Valley Unit No. 1 and No. 2 peak clad temperature determinations which support the reported values in our February 19, 1993 proposed operating license change request. This information is provided in response to a telephone conversation between our respective staffs which occurred on April 12, 1993.

If you have any questions regarding this submittal, please contact Mr. Steve Sovick at (412) 393-5211.

Sincerely,

Low Sterres to J. D. Sieber

Attachment

CC: Mr. L. W. Rossbach, Sr. Resident Inspector Mr. T. T. Martin, NRC Region I Administrator Mr. G. E. Edison, Project Manager Mr. M. L. Bowling (VEPCO)



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## BVPS UNIT 1 LBLOCA

- 1. Analysis of record:
  - BASH analysis described in UFSAR 14.3.2 and 14.3.2.1 (full spectrum analysis).
    - PCT for most limiting break (C<sub>D</sub> = 0.4) = 1918°F (UFSAR, page 14.3-8) [Key Assumptions: 10% steam generator tubes plugged - UFSAR 14.3.2.2 first paragraph]
  - Partial BASH analysis described in UFSAR 14.3.2.2 (re-analyzed limiting break only)
    - PCT calculated as 2149°F (UFSAR, page 14.3-10) [Key Assumptions: 20% steam generator tubes plugged UFSAR 14.3.2.2]
- 2. Most recently reported change to 1 above prior to 2/19/93:
  - None
- 3. Specific changes since 2 above prior to 2/19/93:
  - Fuel rod backfill initial pressure uncertainty<sup>(1)</sup> 2°F<sup>(2)</sup>
    RCS Tavg uncertainty 2°F<sup>(3)</sup>
- (1) Letter ET-NRC-91-3647, 12/20/91, Interim Report No. 91-021, S.R. Tritch (Westinghouse to NRC).
- (2) Interim issue under Westinghouse evaluation.
- (3) Proposed license change request dated 2/19/93; plant specific uncertainty calculation.

## BVPS UNIT 1 SBLOCA

1. Analysis of record:

· NOTRUMP analysis described in UFSAR 14.3.1

PCT for most limiting break (3" break) = 1802°F (UFSAR, page 14.3-4) [Key Assumptions: as described in UFSAR 14.3.1]

- 2. Most recently reported change to 1 above prior to 2/19/93:
  - · Annual 10 CFR 50.46 report dated July 20, 1992
    - PCT reported as 2010°F
- 3. Specific changes since 2 above prior to 2/19/93:

8	Auxiliary feedwater flow rate change	175°F(1)
4	Remove penalty for SBLOCA rod internal	-53°F(2)
	pressure initial condition assumption	
*	Auxiliary feedwater pump start signal	6°F
	assumption change	
*	Burst/Blockage SPIKE interim penalty	204°F(3)
÷.,	Burst/Blockage SPIKE increase due to	$10^{\circ}F^{(3)}$
	2/19/93 proposed license change request	
×.	Non-cycle specific power shape/P-bar-HA	-118°F <sup>(4)</sup>
×.	Cycle specific power shape/P-bar-HA	$-42 \circ F(4)$
	PCS Tava uncertainty	E 0 T (5)

- (1) This penalty is assigned as a contingency to accommodate a reduced auxiliary feedwater flow rate, if needed. No credit is currently taken for it.
- (2) This issue is bounded by the Burst/Blockage SPIKE interim issue. These two issues are mutually exclusive. See letter ET-NRC-91-3647, 12/20/91, S. R. Tritch (Westinghouse to NRC), Interim Report No. 91-021.
- (3) This is an interim issue under evaluation. See letter ET-NRC-91-3647 (above), Interim Report No. 91-005.
- (4) Plant specific input values were used rather than the previous more generic, bounding values. This is not a change to the model.
- (5) Proposed license change request dated 2/19/93; plant specific uncertainty calculation.

## BVPS UNIT 2 LBLOCA

1. Analysis of record:

· BART analysis described in UFSAR 15.6.5

PCT for most limiting break  $(C_D = 0.4) = 2120$ °F (UFSAR Table 15.0-9, UFSAR 15.6.5.3.3, page 15.6-18) [Key Assumptions: 5% steam generator tubes plugged - UFSAR Table 15.6-9]

2. Most recently reported change to 1 above prior to 2/19/93:

· Annual 10 CFR 50.46 report dated July 20, 1992.

PCT reported as 2191°F

3. Specific changes since 2 above prior to 2/19/93:

• WREFLOOD structural heat model -25°F(1)

(1) This change was identified by Westinghouse to us at approximately the same time our 2/19/93 submittal was issued and is, therefore, not reflected in item 2 above '2191°F)

## BVPS UNIT 2 SBLOCA

1. Analysis of record:

NOTRUMP analysis described in UFSAR 15.6.5

 PCT for most limiting break (4" break) = 1399°F (UFSAR 15.6.5.3.3, page 15.6-19 and Table 15.6-10) [Key Assumptions: as described in UFSAR 15.6.5]

2. Most recent reported change to 1 above prior to 2/19/93:

· Annual 10 CFR 50.46 report dated July 20, 1992

· PCT reported as 2176°F<sup>(1)</sup>

3. Specific changes since 2 above prior to 2/19/93:

· None

 The summary reported in our 2/19/93 submittal provides an itemization of this reported PCT.