



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated February 1, 1993 (Reference 1), Nebraska Public Power District (the District, NPPD, or the licensee) submitted a request for an amendment revising the Cooper Nuclear Station (CNS) Technical Specifications (TS) to (1) incorporate the NRC Staff position on leak detection per the guidance of Generic Letter (GL) 88-01 (Reference 2) and its supplement (Reference 3), (2) incorporate the NRC Staff position on inservice inspection schedules, methods, personnel, and sample expansion per the guidance of GL 88-01 and its supplement, and (3) make administrative changes wherein certain system names are replaced by system names which are more consistent with those used in other portions of the TS and implementing surveillance procedures.

2.0 EVALUATION

The CNS licensee has proposed several changes to the TS to meet the guidance of GL 88-01 and GL 88-01, Supplement 1, and to make editorial changes. The NRC staff has reviewed the changes and finds them acceptable, as discussed below.

- (1) Limiting Condition for Operation (LCO) 3.6.C.1 is revised by this amendment to state that reactor coolant leakage into the primary containment from unidentified sources shall not increase at a rate greater than 2 gpm within the previous 24 hour period. If this LCO cannot be met, the revised TS requires that the reactor must be in Cold Shutdown within 24 hours. This change imposes a 2 gpm limit on the increase of unidentified reactor coolant leak rate over a 24-hour period, establishes specific operability requirements for drywell sump flow measuring systems, and increases the frequency of Reactor Coolant System (RCS) leakage checks. The licensee states that this change provides more stringent criteria for the early detection of unidentified leakage within primary containment. This additional restriction will enhance the ability to detect leaks in the reactor coolant pressure boundary, thereby reducing the potential for a significant failure of the pressure boundary. This change incorporates additional restrictions into the TS and does not involve the modification or addition of any plant hardware, nor does it involve a change in those plant settings that affect plant operation responses. This change conforms to part 1 of the staff position on leak detection given in GL 88-01, and is, therefore, acceptable.

- (2) LCO 3.6.C.2 is revised by this amendment to limit the outage time of one sump flow measuring system to 24 hours. However, the revised LCO states that if leakage can be quantitatively measured by manually pumping the sump or measuring the difference in sump level, outage time may be extended to a maximum of 30 days. This conforms to alternative staff position (3) of GL 88-01, Supplement 1, and is, therefore, acceptable.
- (3) Surveillance Requirement (SR) 4.6.C.1 is revised by this amendment to require that RCS leakage be checked by the sump and containment atmospheric sampling systems and recorded at least once per shift, not to exceed 12 hours. This additional requirement regarding the drywell sump flow measuring systems will provide added assurance that the sumps will always be available for the early detection of unacceptable leakage during plant operations. This conforms to alternative staff position (1) of GL 88-01, Supplement 1, and is, therefore, acceptable.
- (4) TS 4.6.G, "Inservice Inspection," is revised by this amendment to state that the inservice inspection program for piping identified in Generic Letter 88-01, shall be performed in accordance with the staff positions on schedule, methods, personnel, and sample expansion, included in Generic Letter 88-01. The licensee notes that this change incorporates additional restrictions into the TS and does not result in any plant modifications or change in plant hardware. The augmented inservice inspection program, for piping identified in Generic Letter 88-01 does not affect plant operations. However, adoption of this augmented inservice inspection program provides added assurance that piping susceptible to Intergranular Stress Corrosion Cracking (IGSCC) will maintain integrity throughout all modes of plant operation. This change conforms to alternative staff position (6) of GL 88-01, Supplement 1, and is, therefore, acceptable.
- (5) A number of editorial changes were made by this amendment to TS pages 135, 137, and 149. These editorial changes make the TS internally consistent and to correct errors. The changes were made to TS Sections LCO 3.6.C, "Coolant Leakage," LCO 3.6.E, "Jet Pumps," LCO 3.6.F, "Recirculation Pump Flow Mismatch," LCO 3.6.G, "Inservice Inspection," SR 4.6.C, "Coolant Leakage," SR 4.6.E, "Jet Pumps," and SR 4.6.G, "Inservice Inspection." These changes involve the replacement of various terms (system names) used to refer to the drywell air sampling system and the sump flow measuring systems with the terms "drywell air sampling system" and "sump flow measuring systems." The purpose of this change is to utilize system names that are consistent with those used in other parts of the TS and the applicable implementing surveillance procedures. This change is administrative and does not involve a change in plant operations, plant modification, or a change in plant hardware. Also, appropriate modifications were made to TS Bases 3.6.C, 4.6.C, 3.6.D, and 4.6.D to reflect the changes made to the body of the TS. The NRC staff has reviewed these changes and finds them acceptable because they are administrative in nature.

In summary, the NRC staff has reviewed the TS changes made by this amendment and finds the acceptable for the reasons given above. Accordingly, the TS changes proposed in Reference 1 are hereby approved.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Nebraska State official was notified of the proposed issuance of the amendment. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (57 FR 12264). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

- (1) Letter and enclosures from G. R. Horn, Nebraska Public Power District, to USNRC, dated February 1, 1993, "Proposed Change No. 112 to Technical Specifications, Generic Letter 88-01 Incorporation, Cooper Nuclear Station, NRC Docket No. 50-298, DPR-46."
- (2) NRC Generic Letter 88-01, "NRC Position on IGSCC [Intergranular Stress Corrosion Cracking] in BWR [Boiling Water Reactor] Austenitic Stainless Steel Piping," dated January 25, 1988.
- (3) Supplement 1 to NRC Generic Letter 88-01, "NRC Position on Intergranular Stress Corrosion Cracking (IGSCC) in BWR Austenitic Stainless Steel Piping," dated January 30, 1992.

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