Entergy
Operations

Entergy Operations, Inc.
Route 3 Box 137G
Russellville, AR 72801
Tel 501-964-3100

April 15, 1993

2CAN049304

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Monthly Operating Report

Gentlemen:

Monthly Operating Report statistics for Arkansas Nuclear One, Unit-2, for March, 1993 is attached. This report is submitted in accordance with ANO-2 Technical Specification 6.9.1.6.

Very truly yours,

James J. Fisicaro

James J. Fisicaro
Director, Licensing

JJF/JRH/prg
Attachment

190103

9304190119 930331
PDR ADOCK 05000368
R PDR

IE24 1/1

cc: Mr. James L. Milhoan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One - ANO-1 & 2
Number 1, Nuclear Plant Road
Russellville, AR 72801

Mr. Roby Bevan
NRR Project Manager, Region IV/ANO-1
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

Mr. Thomas W. Alexion
NRR Project Manager, Region IV/ANO-2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

OPERATING DATA REPORT

DOCKET NO: 50-368
DATE: April 2, 1993
COMPLETED BY: M. S. Whitt
TELEPHONE: (501) 964-5560

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: March 1-31, 1993
3. Licensed Thermal Power (MWt): 2,815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

		MONTH	YR-TO-DATE	CUMULATIVE
11.	Hours in Reporting Period	744.0	2,160.0	114,096.0
12.	Number of Hours Reactor was Critical	744.0	2,160.0	86,591.3
13.	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	744.0	2,160.0	84,739.4
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	2,073,261	6,043,549	223,810,352
17.	Gross Electrical Energy Generated (MWH)	688,850	2,008,860	73,639,051
18.	Net Electrical Energy Generated (MWH)	658,506	1,920,720	70,048,839
19.	Unit Service Factor	100.0	100.0	74.3
20.	Unit Availability Factor	100.0	100.0	74.3
21.	Unit Capacity Factor (Using MDC Net)	103.2	103.6	71.6
22.	Unit Capacity Factor (Using DEC Net)	97.0	97.5	67.3
23.	Unit Forced Outage Rate	0.0	0.0	11.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
25. If Shut Down At End of Report Period. Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		12/05/78
INITIAL ELECTRICITY		12/26/78
COMMERCIAL OPERATION		03/26/80

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO:	50-368
UNIT:	Two
DATE:	April 2, 1993
COMPLETED BY:	M. S. Whitt
TELEPHONE:	(501) 964-5560

MONTH March, 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	892
2	891
3	893
4	893
5	893
6	887
7	893
8	891
9	892
10	895
11	895
12	894
13	896
14	896
15	896
16	893
17	895
18	895
19	816
20	894
21	895
22	895
23	896
24	895
25	893
26	892
27	891
28	890
29	888
30	889
31	722
AVGS:	885

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

MARCH 1993

UNIT TWO

The unit began the month of March at 100% power.

At 1043 hours on the nineteenth, a power reduction to 80% was initiated due to fire water intrusion into a Feedwater Turbine cabinet, generating an inadvertent Turbine Generator setback signal. After informing the System Dispatcher of the load reduction and cause, the dispatcher ordered a further derate to 75% at 1150 hours. The unit was released by the dispatcher at 1818 hours on the nineteenth and the unit attained 100% at 2132 hours that same day. The System Dispatcher requested a power reduction to 75% on the thirty-first and a power reduction was commenced at 0047 hours. The unit attained 75% at 0345 hours and while at reduced power a condenser tube leak was located and plugged.

The unit was held at 75% per the dispatcher for the remainder of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR MARCH, 1993

DOCKET NO.	50-368
UNIT NAME	ANO Unit 2
DATE	April 2, 1993
COMPLETED BY	M. S. Whitt
TELEPHONE	501-964-5560

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR[3]	LICENSEE EVENT REPORT #	SYSTEM CODE[4]	COMPENENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
none									

1
F: Forced
S: Scheduled

2
Reason:
A - Equipment Failure (Explain)
B - Maintenance of Test
C - Refueling
D- Regulatory Restrictior
E - Operator Training & License Examination
F - Administration
G - Operational Error
H - Other (Explain)

3
Method:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Continuation
5 - Load Reduction
9 - Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

DATE: March, 1993

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2

2. Scheduled date for next refueling shutdown. March 25, 1994

3. Scheduled date for restart following refueling. May 16, 1994

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   Yes, Technical Specification changes to relocate cycle specific parameters to a Core Operating Limits Report and to make the Technical Specification requirements for Azimuthal Power Tilt consistent with accident analyses.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. August, 1993

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   None.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 565

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   present 988 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   DATE: 1997 (Loss of fullcore offload capability)