Carolina Power & Light Company

Brunswick Nuclear Plant P. O. Box 10429 Southport, N.C. 28461-0429

APR 01 1993

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U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

> BRUNSWICK STEAM ELECTRIC PLANT UNIT 1 DOCKET NO. 50-325 LICENSE NO. DRP-71 LICENSEE EVENT REPORT 1-93-006

Gentlemen:

In accordance with Title 10 of the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is submitted in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

mBrow for

G. C. Warren, Plant Manager Unit 2 Brunswick Nuclear Plant

SE271

GT/gt

Enclosure

cc: Mr. S. D. Ebneter Mr. P. D. Milano BSEP NRC Resident Office

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U.S. NUCLEAR REGULATORY COMMISSIO						ION E II C A R T N	APPROVED OMB NO. 3150-0104 EXPIRES: 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS: FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBE 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20565-0001, AND TO THE PAPERWORK REDUCTION PROJECT (\$150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.							
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NRC FORM 366A (5/92)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104 EXPIRES: 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH IMNBE 77141, U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20565-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3156-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)		PAGE (3)			
Brunswick Steam Electric Plant	05000325	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2	
Unit 1		93	-06-	0		

TEXT iff more space is required, use additional NRC Form 3664 (c) (17)

TITLE

INOPERABLE FIRE BARRIER PENETRATION SEALS

INILAL CONSTICNS

On March 2, 1993, both Units were in Cold Shutdown as part of the outage started April 21, 1992.

The on-going Fire Barrier Inspection program Design Guide (DG-IV.0020) was implemented to establish the methodology for inspection/reinspection and documentation for fire barrier configurations against specified acceptance criteria. This inspection includes visually verifying the integrity of 100% of the fire barrier penetrations, and ensuring the correctness of the Fire Barrier Index Drawings.

EVENT NARRATIVE

The on-going Fire Barrier Inspection program per DG-IV.0020 identified several .'re barrier penetrations with discrepant fire seal applications. Discrepancies were noted for five cable tray penetrations (CB-1-059, CB-1-060, CB-1-094, CB-1-098, CB-1-105) through the roof of the battery rooms. These contain a Delta Maid One Shot seal (insulating cement for piping) which has been tested only for wall seal applications. Four seals (DG-2-192, DG-2-117, DG-2-152, DG-2-076) were identified in the Emergency Diesel Generator Building with indeterminate fire seals. The Fire Barrier Penetration Index showed the seal design as "N/A," but no analysis justifies this designation. The seals were between each of the four EDG cells and their fuel oil cells.

CAUSE OF EVENT

The cause of the inadequate fire seal design is not known but the 100% inspection of fire cause will accure the seals are installed as required.

CORRECTIVE ACTIONS

Fire watches are stationed and will remain in place until the deficient fire seals are restored and the Fire Barrier Inspection program for areas with deficiencies are completed.

Reinspection of 100% of the fire barriers per DG-IV.0020 is continuing with repair of deficiencies to be completed prior to the end of the present Unit 1 outage and the next Unit 2 refueling outage. Currently 91% of Unit 2, 38% of Unit 1, and 79% of the EDG Building have been inspected. This reinspection includes a visual inspection of the fire barrier penetrations to verify their integrity and to ensure the correctness of the Fire Barrier Index Drawings. The reinspection will determine whether the barriers have degraded so as not to be in accordance with acceptance criteria based on original plant design documentation.

An ongoing review is being conducted of the Fire Barrier Inspection program findings to insure the programatic controls needed to prevent the reintroduction of similar deficiencies are in place (Report scheduled for November 30, 1993).

NRC FORM 366A (5/92)

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FACILITY NAME (1)	DOCKET NUMBER (2)		PAGE (3)			
Brunswick Steam Electric Plant	05000325	YEAR	BEQUENTIAL NUMBER	REVISION NUMBER	3	
Unit 1		93	-06-	0		

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SAFETY ASSESSMENT

The safety significance of the deficient seals is minor due to the administrative controls for combusible materials, installed fire detection, and suppression systems in these areas.

PERIONS CTATTER EVENTS

Related items were previously identified in LER 1-92-012 and Special Report 1-92-004.