



Carolina Power & Light Company

Brunswick Nuclear Plant
P. O. Box 10429
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MAR 05 1993

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10CFR50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-93-001

Gentlemen:

In accordance with Title 10 of the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is submitted in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. C. Warren, Plant Manager - Unit 2
Brunswick Nuclear Plant

SFT/

Enclosure

cc: Mr. S. D. Ebnetter
Mr. P. D. Milano
BSEP NRC Resident Office

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME: (1)

Brunswick Steam Electric Plant, Unit 2

DOCKET NUMBER (2)

05000324

PAGE (3)

1

TITLE (4)

Technical Specification Requirements Exceeded When Auxiliary Reactor Building Roof Vent Radiation Monitor Was Inadvertently De-energized

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
02	05	93	93	- 001 -	000	03	08	93	FACILITY NAME	DOCKET NUMBER
										05000
										05000

OPERATING MODE (9)	04	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following)(11)								
		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		
POWER LEVEL (10)	00	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER		
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract and Text)		
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)				
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME

Steve F. Tabor, Regulatory Compliance Specialist

TELEPHONE NUMBER

(919) 457-2178

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDPS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDPS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

On February 5, 1993 at 0245 hours with both units in cold shutdown, the Unit 2 Reactor Building Roof Vent Radiation Monitor, 2-CAC-AT-1264, was declared inoperable to support plant modification design changes to the monitor. The requirements of the radioactive gaseous effluent monitoring instrumentation Technical Specification, allow continued effluent release provided continuous auxiliary sampling is established. Continuous auxiliary sampling was established at approximately 0340 hours. The auxiliary sample pump was energized by power supplied from a 120v receptacle located on the 117' elevation of the Unit 2 Reactor Building and fed through a series of extension cords. At 1648 hours, during a required walkdown to verify sample flow, the extension cord was discovered disconnected. At approximately 1707 hours, power was restored to the auxiliary sample pump and sample flow restored. The loss of power to the auxiliary sample pump occurred because those responsible for supplying power to the auxiliary sample pump did not follow plant procedure. Immediate corrective action included the restoration of power to the auxiliary sample pump. Additional corrective actions include a revision to the auxiliary sampling procedure and a review of the event with appropriate personnel. The safety significance of this event is minimal in that both units were in cold shutdown and no work activities were in progress at the time of the event which could have caused abnormal activity concentrations in the building. A review of the Unit 2 Reactor Building continuous air sample data and effluent radiation levels immediately prior to and following the event revealed that activity concentration levels were normal. Previous similar events have been reported in LERs 1-92-007 and 1-92-025.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20565-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Brunswick Steam Electric Plant Unit 2	05000324	93	- 001 -	000	2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TITLE

Technical Specification Requirements Exceeded When Auxiliary Reactor Building Roof Vent Radiation Monitor Was Inadvertently De-energized

INITIAL CONDITIONS

Units 1 and 2 were in cold shutdown since April 21, 1992 due to an extended maintenance outage. Preparations were in progress to remove wiring associated with the Unit 2 Reactor Building Roof Vent Radiation Monitor iodine and particulate sampling control circuitry in accordance with plant modification PM 92-017.

EVENT NARRATIVE

On February 5, 1993, at approximately 0245 hours, the Unit 2 Reactor Building Roof Vent Radiation Monitor, 2-CAC-AT-1264, was declared inoperable to support modification of the monitor in accordance with plant modification PM 92-017. Technical Specification allows effluent releases to continue provided that a continuous auxiliary sample is established. At approximately 0340 hours, Environmental and Radiation Control (E&RC) personnel placed the auxiliary sampling equipment in service in accordance with plant procedure. At approximately 0900 hours, during a walkdown to verify auxiliary sample flow in accordance with the 8 hour surveillance period required by Technical Specifications, E&RC personnel verified that the auxiliary sample pump was operating satisfactorily. At approximately 1648 hours, during performance of the next required surveillance, E&RC personnel discovered that the auxiliary sample pump was not operating and that the extension cord used to provide power to the pump had been disconnected. At 1707 hours, the cord was reconnected and sample flow restored.

CAUSE OF EVENT

This event occurred because the individuals responsible for providing the 120v electrical power to the auxiliary sample pump did not comply with the procedural requirements for control and use of unattended extension cords, nor did they communicate to craft personnel the need to use a craft controlled extension cord for the purpose of supplying power to significant equipment.

CORRECTIVE ACTIONS

On February 14, 1993 at 1250 hours, auxiliary sampling was terminated and the 2-CAC-AT-1264 returned to operation.

A briefing with appropriate E&RC personnel has been conducted to convey management expectations and the need to ensure procedural compliance regarding the use of extension cords.

E&RC procedure, E&RC-2002, Sampling of Radioactive Airborne Effluent Releases, will be revised by March 31, 1993 to specify the actions necessary for establishing auxiliary radiation monitor sample pump temporary power. Training of appropriate E&RC personnel on the revision to E&RC-2002 will occur as part of the procedure revision process.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNR 7754), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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Brunswick Steam Electric Plant Unit 2	05000324	93	- 001 -	000	3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

SAFETY ASSESSMENT

The safety significance of this event is minimal in that both units were in cold shutdown and those work activities which could have caused abnormal activity in the building were not in progress at the time of the event. A review of Unit 2 Reactor Building continuous air sample data and effluent activity concentrations immediately prior to and following the event revealed that activity levels were normal.

PREVIOUS SIMILAR EVENTS

Previous similar events have been reported in LERs 1-92-007 and 1-92-025.

EIIS COMPONENT IDENTIFICATION

System/Component

EIIS Code

CAC
2-CAC-AT-1264

IK
IK/45