Northern States Power Company

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February 22, 1993

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Response to Request for Additional Information Regarding the License Amendment Request - Administrative Changes Related to Organization, Operations Committee Membership and Working Hours

The attached information is provided in response to the October 5, 1992 NRC letter which transmitted NRC Staff questions related to our License Amendment Request dated February 25, 1991, titled "Administrative Changes Related to Organization, Operations Committee Membership and Working Hours." As a result of our consideration of the Staff questions we agree to make additional changes which are reflected in the attached revised page changes to the Technical Specifications, Appendix A of the Operating Licenses, for Prairie Island Nuclear Generating Plant, Units 1 and 2.

The significant hazards analysis and safety evaluation submitted with the February 25, 1991 License Amendment Request still apply.

This letter contains no new NRC commitments.

Please contact Jack Leveille (612-388-1121, Ext. 4662) if you have any questions related to this letter.

Thomas M Parker
Director
Nuclear Licensing

c: Regional Administrator - Region III, NRC
Senior Resident Inspector, NRC
NRR Project Manager, NRC
J E Silberg

Attachments: Affidavit

Response to Request for Additional Information Regarding the License Amendment Request - Administrative Changes Related to Organization, Operations Committee Membership and Working Hours

Revised Exhibit C - REVISED TECHNICAL SPECIFICATIONS PAGES

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UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT DOCKET NO. 50-282
50-306

REQUEST FOR AMENDMENT TO
OPERATING LICENSES DPR-42 & DPR-60

RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION REGARDING THE
LICENSE AMENDMENT REQUEST DATED FEBRUARY 25, 1991

Northern States Power Company, a Minnesota corporation, requests authorization for changes to Appendix A of the Prairie Island Operating License as shown on the attachment labeled Exhibit C. Exhibit C contains the revised Technical Specification pages.

This letter contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By Thomas M Parker
Thomas M Parker
Director, Nuclear Licensing

On this 22 day of February 1993 before me a notary public in and for said County, personally appeared Thomas M Parker, Director, Nuclear Licensing, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof, and that to the best of his knowledge, information, and belief the statements made in it are true and that it is not interposed for delay.

Judy L. Klapperick

JUDY L. KLAPPERICK
NOTARY PUBLIC-MINNESOTA
ANOKA COUNTY
My Commission Expires Sept. 29, 1997

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Response to Request for Additional Information Regarding the License Amendment Request - Administrative Changes Related to Organization, Operations Committee Membership and Working Hours

Question 1:

Generic Letter 88-06 states that if the organizational charts are proposed to be removed from the TS and added to the Quality Assurance Plan then the licensee should first complete the incorporation of the charts into the QA Plan to ensure that the organization information is included in the appropriate documentation. Please provide a copy of the Quality Assurance Plan showing the incorporation of the appropriate organizational information.

Response:

We have changed the proposed wording from ". . . documented in corporate and plant procedures and the Operational Quality Assurance Plan" to ". . . documented in the Operational Quality Assurance Plan or the Updated Safety Analysis Report." See the attached "Revised Exhibit C - REVISED TECHNICAL SPECIFICATIONS PAGES."

We believe that the combination of the information contained in these two documents provides the necessary organizational detail. The Updated Safety Analysis Report organizational information is contained Chapter 13, Plant Operations, which is in Volume 4. The current revision of the Operational Quality Assurance Plan is Appendix C of the Updated Safety Analysis Report, contained in Volume 5. The Updated Safety Analysis Report is revised and submitted to the NRC periodically. If you desire a separate copy of this information, we will provide it.

Question 2:

Insert "B" for the proposed technical specification changes does not follow the guidance provided in the markup of the Westinghouse standard technical specifications shown in Generic Letter 88-06. Reference to a corporate officer is not sufficient. The staff prefers that the guidance, designating specific titles, be followed. However, if the licensee does not wish to incorporate specific titles into the technical specifications, then the position should be described in such a way that the position can be easily and consistently identified from current organizational charts in the QA plan. For example, "the senior management position on site" may be an adequate definition for the plant manager. Provide a revision incorporating the above guidance for all applicable items.

Response:

The specific titles have been restored and or revised, as appropriate, in the proposed Technical Specification pages. See the attached "Revised Exhibit C - REVISED TECHNICAL SPECIFICATION PAGES."

Question 3:

In Section 6.2-5 B, Operations Committee (OC), part 3, Quorum, appears to allow a quorum to be achieved without a majority of the regular members. Clarify the requirements for a quorum of the Operations Committee.

Response:

The proposed change does not require a majority of the regular members in order to achieve a quorum. That is the purpose of the proposed change. We intentionally have a rather large regular membership in order to obtain balanced and informed reviews. However, by doing this, the quorum requirements, at times, impose a limitation on our ability to obtain timely Operations Committee review (e.g., during periods of time when there are several members who are unavailable). We believe that allowing two alternates to serve and to be counted toward the quorum requirements enhances the ability of the Operations Committee to provide expeditious reviews.

Chapter 13 of the Updated Safety Analysis Report identifies nine regular members of the Operations Committee. By the proposed wording, a quorum would be five members of which two could be alternates and the minimum number of regular members would be three. The minimum number of regular members required by the Westinghouse Standard Technical Specifications current at the time of this original submittal was also three.[1] In addition, the proposed wording was approved for use at Monticello Nuclear Plant in a License Amendment dated September 12, 1989.

In the light of the above considerations, we believe the proposed change is appropriate.

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[1]The present Westinghouse Owners Group Technical Specification does not specify an Operations Committee organization.