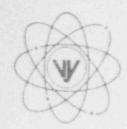
VERMONT YANKEE NUCLEAR POWER CORPORATION



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ENGINEERING OFFICE

February 19, 1993 BVY 93-016 TDL 93-07

United States Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

References:

License No. DPR-28 (Docket No. 50-271) (a)

(b) NRC SRO Exam Administered at Vermont Yankee 02/18/93

Subject:

Comments on NRC SRO Written Exam

Dear Sir:

The following are our comments on the NRC SRO written exam administered on 02/18/93:

Question 47:

In accordance with AP 0894, "Shift Staffing/Overtime Limits" we are required to have two auxiliary operators on shift and two reactor operators (one CRO and one ACRO). We are also required to have four people for the fire brigade. Two of the four fire brigade positions are fulfilled by the auxiliary operators. This would mean that the ACRO is not part of the fire brigade and he would be available to go to the Control Room, if not already there and in accordance with AP 0151, "Resp. & Auth. of Ops. Pers.," page 13 number 10, the ACRO is required to report to the Control Room.

Therefore, answers "A" or "C" should be accepted as correct.

Question 48:

In accordance with AP 0140, "Switching and Tagging" page 5 step 3.a and page 9 step 3.b, when a tag off order is being issued to a switchman he "write(s) the tags off order... and then repeat(s) the order back to the control authority." This repeat back is done "as written" i.e., "verbatim." As supported by the procedure, as we train and as done in actual practice

this communication is "verbatim."

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Therefore, answers "B" or "C" should be accepted as correct.

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Question 58:

A comment was made during pre-exam review that distractor "B" was correct. On the exam that was administered "B" was changed to make it incorrect. However, this still does not make "D" correct. While the reference quoted states that the 950 is chose to "... limits thermal cyclic stress," it is not to "limit thermal stress on the subject SRV." EOP study guide page 7 - 24 indicates 940 is chosen to "minimize RPV pressure/temperature swings."

Therefore, no answer is correct and the question should not be counted.

Question 73:

In reference to the stem of the question and distractor "B", if the mode switch is in RUN and reactor pressure is 780 psig, i.e., < 800 psig, then the MSIV's should have isolated and a scram should have occurred. If the examinee assumes a scram has not occurred, based on the stem of the question, then the plant is in an ATWS condition and initiating an immediate reactor scram is an appropriate action.

Therefore, answers "B" or "D" should be accepted as correct.

Question 07:

While it is true that the maximum level error with drywell temperature at 280 degrees is 38 inches, this magnitude of error exists with a reactor pressure of ~ 100 psia. With drywell temperature at 280 degrees and reactor pressure at 1000 psia, the magnitude of the level error is in the range of 6 - 8 inches. These numbers are based on YAEC-1605, "Reactor Vessel Water Level Error Analysis." Since the stem of the question does not define what reactor pressure is, the examinee could assume 1000 pounds reactor pressure, thereby justifying a six inch level error.

Therefore, answers "B" or "D" should be accepted as correct.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

Mark L. Mervine Training Manager

C: USNRC Region I Administrator
USNRC Resident Inspector - VYNPS
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