



Nebraska Public Power District

GENERAL OFFICE
P.O. BOX 499, COLUMBUS, NEBRASKA 68602-0499
TELEPHONE (402) 564-8561
FAX (402) 563-5551

RECEIVED

1 '91 MAR 20 P7:44
PUBLIC DOCUMENT ROOM

NLS9100130
March 8, 1991

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Revision to Proposed Change No. 81 to Technical Specifications
Cooper Nuclear Station
NRC Docket No. 50-298, DPR-46

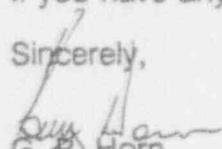
Reference: Letter, G. A. Trevors to USNRC dated July 2, 1990, "Proposed Change No. 81 to Technical Specifications - Deletion of Cycle-Specific Parameter Limits"

Gentlemen:

In the above reference, the Nebraska Public Power District (the District) submitted a proposed change to the Cooper Nuclear Station Technical Specifications that would delete the cycle-specific parameter limits following the guidance contained in Generic Letter 88-16. Subsequent NRC staff review of this application generated several comments regarding the wording of a select number of the proposed Technical Specifications and their bases. These comments are resolved by certain clarifications and modifications made to various proposed Technical Specification pages that were submitted in the reference. The revised pages incorporating these changes are attached for consideration. The District considers that the no significant hazards consideration contained in the reference to still be valid.

If you have any questions, please call.

Sincerely,


G. A. Horn

Nuclear Power Group Manager

GRH/grs:sm
Attachment

cc: Regional Administrator
USNRC - Region IV
Arlington, Texas

NRC Resident Inspector Office
Cooper Nuclear Station

9103120213 910308
PDR ADOCK 05000298
P PDR

A001
11

1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the specifications may be achieved.

A. Thermal Parameters

1. Critical Power Ratio (CPR) - The critical power ratio is the ratio of that assembly power which causes some point in the assembly to experience transition boiling to the assembly power at the reactor condition of interest as calculated by application of an NRC approved critical power correlation.
 2. Maximum Fraction of Limiting Power Density - The Maximum Fraction of Limiting Power Density (MFLPD) is the highest value existing in the core of the Fraction of Limiting Power Density (FLPD).
 3. Minimum Critical Power Ratio (MCPR) - The minimum critical power ratio corresponding to the most limiting fuel assembly in the core.
 4. Fraction of Limiting Power Density - The ratio of the linear heat generation rate (LHGR) existing at a given location to the design LHGR for that bundle type. Design LHGR's for each type of fuel are specified in the Core Operating Limits Report.
 5. Transition Boiling - Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.
- B. Alteration of the Reactor Core - The act of moving any component in the region above the core support plate, below the upper grid and within the shroud. Normal control rod movement with the control rod drive hydraulic system is not defined as a core alteration. Normal movement of in-core instrumentation is not defined as a core alteration.
- C. Cold Condition - Reactor coolant temperature equal to or less than 212°F.
- D. Design Power - Design power means a steady-state power level of 2486 thermal megawatts. This is 104.4% of Rated Power (105% of rated steam flow).
- E. Engineered Safeguard - An engineered safeguard is a safety system the actions of which are essential to a safety action required to maintain the consequences of postulated accidents within acceptable limits.
- E.A Dose Equivalent I-131 - The DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcurie/gram) which alone would produce the same thyroid dose if inhaled by an adult as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The dose equivalent I-131 concentration is calculated by: $\text{equiv. I-131} = (I-131) + 0.0096 (I-132) + 0.18 (I-133) + 0.0025 (I-134) + 0.037 (I-135)$.
- E.B Exhaust Ventilation Treatment System - An EXHAUST VENTILATION TREATMENT SYSTEM (EVTS) is a system intended to remove radioiodine or radioactive material in particulate form from gaseous effluent by passing exhaust ventilation air through charcoal absorbers and/or HEPA filters before exhausting the air to the environment. An EVTS is not intended to affect noble gas in gaseous effluent. Engineered Safety Feature (ESF) gaseous treatment systems are not considered to be EVTS. The Standby Gas Treatment System is an ESF and not an EVTS. EVTS are specifically identified in ODAM Figure 3-1.