



Arkansas Power & Light Company
Arkansas Nuclear One
Route 3, Box 137 G
Russellville, AR 72801
Tel 501 964 3100

April 16, 1990

2CANØ49Ø15

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Stop P1-137
Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 2 Monthly Operating Report for March, 1990 is attached.

Very truly yours,

James J. Fisicaro
James J. Fisicaro
Manager, Licensing

JJF/SAB/lw
Attachment
cc:

Mr. Robert D. Martin
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

9004270304 900334
PDR ADDCK 05000368
R FDC

IEPA
|||

An Entergy Company

OPERATING DATA REPORT

DOCKET NO: 50-368
 DATE: March, 1990
 COMPLETED BY: M. S. Whitt
 TELEPHONE: (501) 964-3743

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: March 1-31, 1990
3. Licensed Thermal Power (Mwt): 2,815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

	<u>MONTH</u>	<u>YR-TO-DATE</u>	<u>CUMULATIVE</u>
11. Hours in Reporting Period	744.0	2,160.0	87,792.0
12. Number of Hours Reactor Was Critical	669.5	1,944.2	64,308.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,430.1
14. Hours Generator On-Line	665.0	1,936.6	62,717.3
15. Unit Reserve Shutdown Hours ..	0.0	0.0	75.0
16. Gross Thermal Energy Generated (MWH)	1,735,876.0	5,116,612.0	163,325,142.0
17. Gross Electrical Energy Generated (MWH)	574,885.0	1,693,205.0	53,650,681.0
18. Net Electrical Energy Generated (MWH)	546,441.0	1,612,132.0	50,989,102.0
19. Unit Service Factor	89.4	89.7	71.4
20. Unit Availability Factor	89.4	89.7	71.5
21. Unit Capacity Factor (Using MDC Net)	85.5	87.0	67.7
22. Unit Capacity Factor (Using DER Net)	80.5	81.8	63.7
23. Unit Forced Outage Rate	10.6	10.3	13.7
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			

25. If Shut Down At End of Report Period. Estimated Date of Startup: _____
26. Units in Test Status (Prior to Commercial Operation): _____

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368
UNIT: Two
DATE: March, 1990
COMPLETED BY: M. S. Whitt
TELEPHONE: (501) 964-3743

MONTH March, 1990

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	888
2	889
3	890
4	359
5	-25
6	-24
7	6
8	588
9	626
10	621
11	604
12	603
13	707
14	880
15	890
16	890
17	891
18	892
19	891
20	888
21	892
22	891
23	893
24	895
25	895
26	894
27	893
28	892
29	890
30	891
31	891

AVGS: 735

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATIONS SUMMARY

MARCH 1990

UNIT TWO

The unit began the month operating at 100% full power (FP).

On the fourth at 0812 hours, a power descent was commenced due to noise in the "D" Core Protection Calculation (CPC) channel. At 1144 hours the unit was taken off line to trouble shoot the source of the noise. The noise source was determined to be caused by a faulty surge capacitor on a Reactor Coolant Pump (RCP). Following replacement of surge capacitors on all four RCPs, the unit was placed back on line on the seventh at 1842 hours. The unit obtained a power level of 70% on the eighth at 0324 hours, where it was held due to a lack of demand on the system. During the power hold a condenser tube leak was suspected due to elevated sodium concentrations in the Condenser/Feedwater System. Following the search for the source of the sodium in-leakage and the release from the power hold by the dispatcher, the unit recommenced a power escalation on the thirteenth at 1034 hours.

On the fourteenth at 0355 hours, the unit attained 100% FP and remained at that level through the end of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR MARCH, 1990

DOCKET NO.	50-368
UNIT NAME	Two
DATE	March, 1990
COMPLETED BY	M. S. Whitt
TELEPHONE	501-964-3743

<u>No.</u>	<u>Date</u>	<u>Type¹</u>	<u>Duration (Hours)</u>	<u>Reason²</u>	<u>Method of Shutting Down Reactor³</u>	<u>Licensee Event Report #</u>	<u>System Code⁴</u>	<u>Component Code⁵</u>	<u>Cause & Corrective Action to Prevent Recurrence</u>
90-01	900304	F	79	A	1	2-90-005	AB	CAP	The unit was taken off line due to noise on the "D" CPC channel. The source of the noise was determined to be a faulty surge capacitor on a RCP. Before the unit was placed back on line, the surge capacitors on all four RCPs were replaced.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training &
License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation
5-Load Reduction
9-Other

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
1022)

5
Exhibit I - Same Source

DATE: March, 1990

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. February 1991
(Beginning of Cycle 8 criticality was 11/18/89)
3. Scheduled date for restart following refueling. April, 1991
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

None Expected. Reload fuel design is in progress.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. None Required
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

To obtain the presently planned cycle 8 length of 420 EFPD, it will be necessary to raise the current peak rod burnup limits. A report justifying an increase was submitted in July, 1989.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 421
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 988 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1996 (Loss of fullcore offload capability)