REACTOR COOLANT SYSTEM SURVEILLANCE REQUIREMENTS (Continued)

- b. The first inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
 - All nonplugged tubes that previously had detectable wall penetrations (>20%), and
 - Tubes in those areas where experience has indicated potential problems.
- c. The second and third inservice inspections may be less than a full tube inspection by concentrating (selecting at least 50% of the tubes to be inspected) the inspection on those areas of the tube sheet array and on those portions of the tubes where tubes with imperfections were previously found.
- d. Tubes in specific limited areas which are distinguished by unique operating conditions and/or physical construction may be excluded from random samples if all such tubes in the specific area of a steam generator are inspected with the inspection result classification and the corresponding action required as specified in Table 4.4-6. No credit will be taken for these tubes in meeting minimum sample size requirements. Degraded or defective tubes found in these areas will not be considered in determining the inspection results category as long as the mode of degradation is unique to that area and not random in nature.

The results of each sample inspection shall be classified into one of the following three categories:

Category	Inspection Results				
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.				
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.				
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.				

Note: In all inspections, previously degraded tubes whose degradation has not been spanned by a sleeve must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- 4.4.5.3 <u>Inspection Frequencies</u> The above-required inservice inspections of steam generator tubes shall be performed at the following frequencies:
 - a. The first inservice inspection shall be performed after 6 Effective Full Power Months, but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under all volatile treatment (AVT) conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category, or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
 - b. If the inservice inspection of a steam generator, conducted in accordance with Table 4.4-2 and/or Table 4.4-6 requires a third sample inspection whose results fall in Category C-3, the inspection frequency shall be reduced to at least once per 20 months. The reduction in inspection frequency shall apply until a subsequent inspection demonstrates that a third sample inspection is not required.
 - c. Additional unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 and/or Table 4.4-6 during the shutdown subsequent to any of the following conditions:
 - Primary-to-secondary tube leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2.
 - 2. A seismic occurrence greater than the Operating Basis Earthquake,
 - A loss-of-coolant accident requiring actuation of the engineered safeguards, or
 - 4. A main steam line or feedwater line break.

4.4.5.4 Acceptance Criteria

- a. As used in this Specification:
 - Tubing or Tube means that portion of the tube or sleeve which forms the primary system to secondary system pressure boundary.
 - Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings

or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.

- Degradation means a service-induced cracking, wastage, wear, or general corrosion occurring on either inside or outside of a tube.
- 4. <u>Degraded Tube</u> means a tube containing imperfections ≥ 20% of the nominal wall thickness caused by degradation except where all such degradation has been spanned by the installation of a sleeve.
- <u>% Degradation</u> means the percentage of the tube wall thickness affected or removed by degradation.
- 6. <u>Defect</u> means an imperfection of such severity that it exceeds the plugging/sleeving limit except where the imperfection has been spanned by the installation of a sleeve. A tube containing a defect in its pressure boundary is defective. Any tube which does not permit the passage of the eddy-current inspection probe shall be deemed a defective tube.
- 7. Plugging/Sleeving Limit means the imperfection depth at or beyond which the tube shall be restored to serviceability by the installation of a sleeve or removed from service because it may become unservice be prior to the next inspection and is equal to 40% of the nominal tube wall thickness. No more than five thousand sleeves may be installed in each steam generator.
- 8. <u>Unserviceable</u> describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.
- 9. <u>Tube Inspection</u> means an inspection of the entire steam generator tube as far as possible.
- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug or sleeve all tubes exceeding the plugging/sleeving limit and all tubes containing through-wall cracks) required by Table 4.4-2 (and Table 4.4-6, if the provisions of Specification 4.4.5.2.d are utilized).

4.4.5.5 <u>Reports</u>

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged and tubes sleeved in each steam generator shall be reported to the Commission within 15 days.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. The complete results of the steam generator tube inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following the completion of the inspection. This Special Report shall include:
 - 1. Number and extent of tubes inspected.
 - Location and percent of wall-thickness penetration for each indication of an imperfection.
 - 3. Identification of tubes plugged and tubes sleeved.
- c. Results of steam generator tube inspections which fall into Category C-3 shall be reported to the Commission pursuant to 10 CFR 50.72 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence pursuant to 10 CFR 50.73.
- 4.4.5.6 The steam generator shall be demonstrated OPERABLE by verifying steam generator level to be within limits at least once per 12 hours.

STEAM GENERATOR TUBE INSPECTION

	IST SA	MPLE INSPECTION	2ND SAMPLE INSPECTION		ORD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	T
	C-1	None	N/A	N/A		Action Required
	C-2	Dlug on alast to			N/A	N/A
		Plug or sleeve defective tubes and inspect additional 25 tubes in this SG	C-1	None	N/A	N/A
			tube addi	Plug or sleeve defective tubes and inspect additional 4S tubes in this SG	C-1	None
					6-5	Plug or sleeve defective tubes.
			C-3	Perform action for C-3 result of first sample.	C-3	Perform action for (-: result of first sample.
					N/A	N/A
	C-3	Inspect all tubes in this SG, plug or sleeve defective tubes, and inspect 2S tubes in each other SG Notify NRC pursuant to 10 CFR 50.72.	All other SGs are C-1	None	N/A	N/A
			So e SGs C-2 but no additional SGs are C-3	Perform action for C-2 result of second sample.	N/A	N/A
	S		SG is C-3	Inspect all tubes in each 56 and plug or sleeve Jeferire tubes. Notify NRC pursuant to 10 CFR 50.72.	N/A	N/A

S = 3N% Where N is the number of steam generators in the unit and n is number n of steam generators inspected during inspection.

TABLE 4.4-6
SPECIFIC LIMITED AREA INSPECTION

		LE INSPECTION OF A IC LIMITED AREA"	2NJ SAMPLE INSPECTION OF A "SPECIFIC LIMITED AREA"		
Sample Size	Result	Action Required	Result	Action Required	
100% of Area in both SGs	C-1	None	N/A	N/A	
	C-2	Plug or sleeve defective tubes.	N/A	N/A	
	C-3	Plug or sleeve defective tubes.	N/A	N/A	
100% of Area in one SG	C-1	None	N/A	N/A	
	C-2	Plug or sleeve defective tubes and inspect 100% of corresponding area in other SG.	C-1	None	
			C-2	Plug or sleeve defective tubes.	
			C-3	Plug or sleeve defective tubes.	
	C-3	Plug or sleeve defective tubes and inspect 100% of corresponding area in other SG.	C-1	None	
			C-2	Plug or sleeve defective tubes.	
			C-3	Plug or sleeve defective tubes.	

system and the secondary coolant system (primary-to-secondary leakage = 1 GFM). Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 1 GPM can be detected by monitoring the secondary coolant. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged.

Operational experience has shown that tube defects can be the result of unique operating conditions and/or physical arrangements in specific limited areas of the steam generators (for example, tubes adjacent to the open inspection lane or tubes whose 15th tube support plate hold is not broached but drilled). A full inspection of all of the tubes in such specific limited areas will provide complete assurance that degraded or defective tubes in these areas are detected. Because no credit is taken for these distinctive tubes in the constitution of the first sample or its results, the requirements for the first sample are unchanged. This requirement is essentially equivalent to and meets the intent of the requirements set forth in Regulatory Guide 1.83, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes", Rev. 1, July 1975, and does not reduce the margin of safety provided by those requirements.

Wastage-type defects are unlikely with proper chemistry treatment of the secondary coolant. However, even if a defect should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging or sleeving will be required for all tubes with imperfections exceeding the plugging/sleeving limit of 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness. For examination of the sleeved steam generator tubing at the lower sleeve end, the indications will be compared to those obtained during the baseline sleeved tube inspection. Significant deviations between these indications will be considered sufficient evidence to warrant designation as a degraded tube.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be promptly reported to the Commission pursuant to Specification 6.9.1 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

The steam generator water level limits are consistent with the initial condition assumptions in the FSAR. The maximum steam generator level, as a function of steam superheat, is sufficient to assure a mass inventory of less than or equal to 62,600 lbm in the event of a main steam line rupture.