OCT 12 1984

J. Ayer. info & file

The Honorable Marilyn Lloyd Chairman, Subcommittee on Energy Research and Production Committee on Science and Technology United States House of Representatives Washington, DC 20515

Dear Madam Chairman:

In response to your letter of September 20, 1984 I am pleased to supplement the record of the July 26, 1984 hearing on the Status of the Technical Infrastructure Required to Support Domestic Food Irradiation. My response to the list of questions submitted with your letter is attached.

Sincerely,

Original Signed by D. R. Chapell

PRICHARD E. Cunningham, Director Division of Fuel Cycle and Material Safety

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Enclosure: As stated

DISTRIBUTION RECunningham	FCMC r/f			
NBassin	NMSS r/f			
BSinger	FC Central File			
JDe1Med1co	DNussbaumer			
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CA				
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EDO - 14819	DMorris			
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GCunningham	SECY - 84-0942			
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**telephonic concurrence by JMapes for 3 GCunningham

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RESPONSES TO ADDITIONAL QUESTIONS

1. QUESTION:

Does the chemical form of the Cesium in DOE's current capsules -- Cesium Chloride (CsCl), which is a liquid and is also very scluble in water -- present any safety problems? How do the main safety concerns in licensing use of a Cs-137 radiation source compare to the concerns in licensing a Cobalt-60 source?

ANSWER:

The cesium chloride (CsCl) contained in the DOE's capsules is a solid. The main safety issues related to these capsules are potential attack by the cesium salt on capsule containment and solubility if released. The DOE has conducted corrosion examinations to confirm the adequacy of the capsule to maintain its integrity and is conducting further tests to determine the effects of thermal cyclings, as would be encountered in use of the capsules in a wet-storage, dry irradiation operation, on the capsule. If the test data demonstrate containment integrity, we believe that the capsules will be adequate for use in irradiators.

The Cobalt-60 contained in currently used capsules is in the form of soild metal seeds which are only slightly soluble in water. The Cobalt-60 capsules have been used several years in irradiators and have a good record of integrity.

Regardless of capsule integrity, backup systems are required to detect leakage. Due to the very high solubility of CsCl compared to metallic Cobalt-60, special procedures are contemplated to manage leakage in the unlikely event that it occurrs.

The safety considerations for operation of an irradiator containing either Cesium-137 or Cobalt-60 are the same.

2. QUESTION:

Would it not be prudent from a safety concern to require the same standards for maintaining an operating license for a cobalt wet storage irradiator?

ANSWER:

Once the integrity and adequacy of the capsules used in an irradiator have been demonstrated, the safety considerations are essentially the same for irradiators containing either cesium chloride or cobalt metal in the capsules. As noted above, the management of CsCl leakage in the event that it occurs requires some additional precautionary procedures due to its solubility.

3. QUESTION:

What manner of waste disposal is currently required for large radiation sources? Do you have an idea of what the cost of disposal is (or might be) on a cost per curie of radiation basis? What process does the NRC use to assure that radiation sources will be disposed of properly?

ANSWER:

Most of the large irradiators in the United States use Cobalt-60 sources purchased from Atomic Energy Canada Limited (AECL). Under present arrangements, most of these sources are replaced and the spent sources are reclaimed by AECL. Cesium chloride sources in the form of WESF capsules are supplied by DOE. DOE has stated that it wil' take back spent capsules for storage. Requirements for disposal of waste by an individual licensee are set forth in the Commission's regulation 10 CFR Part 20, "Standards for Protection Against Radiation," for sources of the type used in irradiators. The regulation permits transfer only to persons specifically licensed to possess the sources such as operators of commercial land burial facilities. The operations of a commercial waste burial ground are governed by the Commission's Regulation 10 CFR Part 61, "Licensing Requirements for Land Disposal of Radioactive Waste," or equivalent regulations by an Agreement State.

Charges by commercial land burial facilities for disposal of wastes are based on package volume and the extent of special handling. For waste of low activity current charges at burial grounds range from \$17.00 to \$30.00 per cubic foot of package volume. Based on our discussions with a burial ground operator, the cost of disposal of very large sources (about 30,000 Ci) could be as high as \$3000 per cubic foot of package volume.

4. QUESTION:

Does the NRC know the current location and condition of all radiation sources that are in the United States? What about sources fabricated in the United States and exported to other countries? Are you confident that the situation that recently occurred in Mexico -- where a cobalt radiation source was relegated to a junkyard and wound up exposing numerous individuals to excessive radiation levels -- was a fluke and is not likely to happen again? Has the NRC taken any actions that could prevent another such occurrence?

ANSWER:

The specific licenses issued by the NRC specify the radiation sources which may be possessed by licensees and specify the location at which the sources may be used. Specific licensees who distribute radiation sources to general licensees are required to report the name and location of persons to whom the sources are sent. Although the NRC does not know the condition of radiation sources at any given time, licenses which provide for possession and use of the sources require periodic leak testing of them. If there is any leakage of radioactive material from a source, the licensee is required to report the leakage to the NRC and take immediate steps to correct the problem. The NRC conducts periodic inspections of persons licensed to possess radiation sources to determine if the licensee's program is in compliance with NRC regulations and the terms and conditions of the licensees.

With respect to radiation sources fabricated in the United States and exported to the other countries, the NRC does not have knowledge of the location and condition of the sources. The NRC staff is analyzing the issues related to notification of foreign governments of the export of radioactive sources to persons in foreign countries.

Although we believe the incident which occurred in Mexico to be a unique situation, the following actions are being taken to minimize other occurrence or the effects of such an occurrence.

- A study under contract with Aerospace Corporation has been initiated to assess the feasibility of detecting unauthorized radioactive material accidentally contained in items that enter the United States through commercial import channels.
- Radiation detection instruments have been provided to each of the 22 U.S. Customs port of entry locations along the U.S./Mexico border and Customs agents have been instructed in the use of the instruments.
- The NRC is developing plans to provide prompt response to incidents such as the Mexican incident. The response plan will incorporate interactions with other Federal agencies and State governments.

RESPONSES TO MINORITY QUESTIONS

1. QUESTION:

What amount of effort is necessary to reach an agreement with a state relative to material irradiation? How soon do you think the remaining 23 states will reach such agreement with NRC?

ANSWER:

Section 274 of the Atomic Energy Act, as amended, provides a mechanism which permits states to assert regulatory authority over certain categories of radioactive materials - byproduct materials (radio-isotopes), source materials (the raw materials of Atomic Energy) and small quantities of special nuclear material. Irradiators typically use byproduct materials. The criteria that states wishing to enter into such agreements must meet are provided in a Commission Policy Statement published in the Federal Register (46 FR 7540, January 23, 1981; 46 FR 36969, July 16, 1981; and 48 FR 33376, July 21, 1983). After an agreement is signed, the Act requires periodic reviews by NRC of Agreement States programs. Guidelines for this purpose have been issued by the Commission as a Policy Statement (see 46 FR 59341, December 4, 1981).

The amount of effort needed to enter into an agreement is a function of several variables including the scope of the proposed agreement, the number of NRC licenses that will be transferred to the state and the amount of preparatory work that is needed. Preparatory work includes drafting and enactment of enabling legislation, recruitment and training of staff to carry out the program, drafting and promulgation of regulations, and development of administrative and technical procedures. Supporting documentation for the Governor's request for the agreement showing that the program will meet the criteria must also be prepared. Typically, once a decision is made to seek status as an Agreement State, preparations for an agreement take one to two years.

Whether or not a state seeks an agreement is a matter of choice for the state. At the present time, NRC is negotiating agreements with Iowa and Illinois. Pennsylvania is actively working toward an agreement. While timeframes have not been firmly set, we believe that agreements with Iowa and Illinois are possible by late 1985 or early 1986. Other states are presently examining the possibility of entering into an agreement; these include Virginia, Massachusetts and Wisconsin.

2. QUESTION:

In your experience, what is the public perception of food irradiation?

ANSWER:

We have not dealt with the general public on the matter of food irradiation. The Public Health Service, Department of Health and Human Services, published a proposal in the Federal Register on February 14, 1984, on the use of radiation sources for the treating of food and is analyzing public response on this issue.

3. QUESTION:

If the individual states are the irradiation regulators, what role do you anticipate for NRC relative to that technology in the future?

ANSWER:

Irradiators are licensed by both the NRC and the Agreement States. The NRC has offered to provide technical assistance to Agreement States in the review of applications for large irradiators. The NRC has dealt directly with the Department of Energy on the matter of the use of the DOE Cesium-137 capsules in irradiators. The regulatory position developed by the NRC, on the basis of DOE experience and our licensing practices, is provided to the Agreement States for their adoption.

For future technology, which might be licensed by an Agreement State, we anticipate analyzing the technology for safety considerations and providing guidance as might be appropriate.



Department of Energy Washington, D.C. 20545

JUL 2 5 1984

Mr. Richard E. Cunningham, Director Division of Fuel Cycle and Material Safety U.S. Nuclear Regulatory Commission Silver Spring, MD

Dear Mr. Cunningham:

As agreed in our July 10, 1984, meeting regarding the No. ear Regulatory Commission's (NRC) licensing of the cesium 137 Waste land lation and Storage Facility (WESF) capsule in irradiators, please find enclosed the June 1984 report PNL-5170, "A Review of Safety Issues That Pertain to the Use of WESF Cesium Chloride Capsules in an Irradiator," which this office is submitting in support of our licensing request.

As discussed, the Department of Energy (DOE) believes that the test data collected to date, especially that from the 6 years operating experience with the Sandia Irradiator for Dry Sewage Solids, should furnish ample evidence that an NRC license for similar irradiators utilizing wet load, dry storage, and dry operation design concept is warranted.

With respect to wet load and storage and dry operation irradiators, we believe that considerable technical information exists from 10 years operation of WESF that are applicable to the licensing process. In addition, we have begun an accelerated thermal cycling tests of two WESF capsules. This evaluation should be completed in 6 months. DOE recommends that NRC license the first wet/dry facility on the basis that:

- (1) a capsule would be removed annually from one of each type of irradiator and subjected to destructive evaluation to confirm that operation of these facilities does not result in conditions beyond those that have previously demonstrated a high degree of integrity of the thick-walled, doubly-encapsulated WESF cesium capsules:
- (2) irradiator operating limits be established well within demonstrated integrity limits, e.g.,

wet load, dry storage, dry operation irradiator

wet load, wet storage, dry operation irradiator operating/storage temperature
limit: 23000C

operating surface temperature limit: 2300°C

thermal cycle limit: 12,000 cycles air to water 200°C maximum delta T

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I appreciate the opportunity that we had on July 10, 1984, to discuss with you and your staff the safety issues and related data. If you have any questions, please contact me accordingly.

Sincerely,

John J. Jicha, Jr., Director R&D and Byproducts Division Office of Defense Waste and Byproducts Management

Enclosure

NOTES ON HEETING WIDDE PERSONNEL 10 JULY 1984

ATTENDING:

BILL M. MULLEN, ALBUQUERQUE OPERATIONS OFFICE
GARTH TINGEY, BATTELLE MORTHWEST

BILL REMINI, DOE HEADQUARTERS

JOHN JICHA, "

""

DICK CUNNINGHAM, NMSS: NRC

NATE BASSIN, "

""

JIM AYER "
""

I. WESF CAPSULE TESTING
A. DOE'S PROBRAM FOR, AND RESULTS OF:

- 1. COMPATABILITY TESTING
 - a. LONG-TERM COEROSION TESTING WAS DISCUSSED BY GARTH
 - P. THERMAL CYCLING WAS DISCUSSED BY BILL MCMULLEN. TESTING

 DOES NOT INCLUDE DETERMINING BY TEST THE COMBINED

 EFFECTS OF THERMAL CYCLING AND CORRESION. ONLY THE

 OUTER CAPSULE WILL BE STUDIED @ 4400 CYCLES OBTAINED

 OVER A PERIOD OF ~ G MONTHS (2 CAPSULES). SEE FIGURES.
 - · GARTH TINGEY OFFERED THAT PERHAPS ENOUGH INFORMATION

 EXISTED, OR WAS BEING OBTAINED, TO DERIVE A TEMPERATURE
 CORROSION PATE RECATIONSHIP.
- 3. JIM AYER INFORMED THAT THE NEO WOULD LIKE TO SEE THE THERMAL CYCLING EFFECT ON CORROSION PATE TREATED.

 A DISCUSSION TO NO CONCLUSION FOLLOWED.
 - 4. BILL MC MULLEN DISCUSSED SWELLING CHARACTERISTICS SOME EXPERIMENTS AND A MATHEMATICAL ANALYSIS WERE
 DONE. THE RESULTS WERE USED TO SCREED OUT HEAVILY ALL
 LOADED CAPSULES FOR REENCAPSULATION AT ORNL.

18. JOHN JICHA WILL PROVIDE DOE VIEWS ON ACCEPTABLE CAPSULE DEHAVIOR AND CONDITIONS FOR USE IN A LETTER TO BE WRITTEN IN THE NEXT WEEK.

II. DEMONSTRATION FACILITIES

- A. DOE'S PREGRAM IS AIMED AT COMMODITIES TREATMENT AND DID DUDGE ACCEPTABILITY, ACCORDING TO JOHN JICHA, AND NOT AT CAPSULE PERFORMANCE, FOR WEI STORAGE / DAY IRRADIATION . JIM AYER ALLOWED THAT NEC MAY REQUIRE SAMPLING AND EXAMINATION OF CAPSULES TO MONITOR PERFORMANCE. DOE SEEMED XUKEWARM TO THIS IDEA; BILL REMINI PLEADED BUDGET UNCERTAINTIES; JOHN JICHA ALLOWED THAT DOE HAS A LONG-TERM COMMITMENT HERE AND THAT FOR THIS DURPOSE FUNDING MAY BE DEDICATED.
 - 1. Sidss. BILL Mª MULLEN INDICATES THAT CORRESION STUDY

 RESULTS AND SIDSS OPERATION MAY INDICATE THAT, FOR

 DRY STORME DRY IRRAPIATION, YENTICATION RATES SHOULD

 BE SUCH THAT ONTER DAPSULE WALL TEMPERATURE BE.

 MAINTAINED BELOW 100°C. HE FURTHER SAID THAT HE

 FELT ENOUGH TESTING DATA AND OPERATING EXPERIENCE

 EXSTED TO YELD HIGH CONFIDENCE WITHE SIDSS AS A SUCCESSFUL

 DEMONSTRATION OF THE DRY STORAGE / DRY IRRADIATION MODE.
 - 2. CACS THE DEMONSTRATION OF AGRICULTURAL COMMODITIES
 IRRADIATION HAS JUST ENTERED THE DESIGN/ONSTRUCTION
 THASES. THIS DIROUGHT USES THE DRY IRRADIATION/WET

 THOTRAGE MODE OF OPERATION. GARTH TINGEY OFFERED
 THAT, BASED ON ANALYTICAL WORK, LITTLE OR NO ST. ESS
 WAS EXPECTED DUE TO THERMAL CYCLING OF WEST
 CAPSULES.

- · BILL ME MULLEN VENTURED THAT HE FELT NOT ENDENNE IN

 DRYPH OR EXPERIENCE EXISTED TO SUILD CONFIDENCE IN

 THE COME-TERM PERFORMANCE OF WEST CAPSULES IN THE

 DRY IRRADIATION / WEST STORAGE MODE. JIM HYER REALW

 STATED THAT MRC: MMSS WOULD LIKE TO SEE EXPERIMENTS

 AND ARADIATION THAT APPRESSED THE CONDINED EFFECTS OF

 OUR EDSION AND THERMAL OYCUME (FORTIONE).
- B. JOHN JICHA WILL PROVIDE DOE VIEWS ON ACCEPTABLE
 PERFORMANCE OF DEMONSTRATION FACILITIES.

B. JOHN JOHR BILL MCMULEN SAID THAT OBJERS FOR!

OSCI IN WEST CAPBULES HAD BEEN RECEIVED AS FOLLOWS:

JOTECH 12 MC;

PADIATION STERILIZERS 40 MC;

PADIATION TECHNOLOGY 15 MC;

CER 0.25 MC;

AECL 20 MC;

TOTAL 87.25 MC;

AND THAT DOE INTENDED TO RESERVE. 20 MC: FOR EXPERIMENTATION, DEMONSTRATION, ETC. THE OFDERS AND RESERVE OREATED A SHORTFALL OF ABOUT 30 MC: DOE'S SOLUTION WOULD BE TO NOT FILL THE ORDER FROM AECL AND PROVIDE PADIATION STERILIZERS WITH 30 MC: INSTEAD OF THE 40 MC: ORDERED. JOHN JICHA ALSO STATED THAT THE \$0.10 / C: OFFER FOR CFCL WOULD BE WITHDRAWN BY OF 20 JULY 1984.

TO DETERMINE THEIR INTEREST IN COCL AT "1.00/CI. ON
THE SUBJECT OF FUTURE AVAILABILITY " 40MC: IS AVAILABLE AT
HAMFORD AND ANOTHER 100 MO: AT SAVANNAM RIVER. THERE
IS NO CONVERSION FACILITY AT SAVANNAM RIVER SO TRANSFORT
TO HAMFORD IS A POSSIBILITY BEING STUDIED. THE TIMETABLE
FOR CONVERSION OF AVAILABLE COCL WAS PROFERRED AS 1987-88
FOR HAMFORD-STORED MATERIAL AND 1990 FOR MATERIAL STORED
IN SAVANNAM RIVER.

II. DIRECTION OF FUTURE EFFORT - JOHN JICHA WILL PROLIDE DOE'S VIEWS IN A LETTER TO BE WRITTEN IN THE WEXT WORK.

PNC 18-20 JULY 1984

ADD JEA TO DISTN OF TESTING REPORTS ON WEST CAPSUES: -

JANDIA 23-24 JULY 1984

10 JULY

BILL Mª MULLEN 846-5236
GARTH TINGEY 375-2419
BILL REMINI
JOHN JICHA
MATE BASSIN
DICK CUNNINGHAM
JIM ATER

III.

104/0: - 1.00/0: ON FUTURE (PURIFIED) CSC. (J.J.) LETTER TO THAT EFFECT IN JULY '84 40 MC: HANFORD, REDESIGN OF ENCAPSULATION. WEADONS PRODUCTION PRIORITIES PRECLUDE CO PRODUCTION. KERIESTED Could Deriver IDTECH 12 MC: IZ MC: RADIATION STERILIZERS 40 MC: - 30 LIC: RADIATION TECHNOLOGY 15 UC: 15 MO; AECL SPOET SHORTHE 30 HO: AVOILACLE ~ 77 MC:

BUSS CARR TO NEC EDELY 85 QUID HANDLE 1 MC:

100 MC. SAVANNAM RIVER '87-88

1 MC:/4e OSCL FROM IDONO COP.

STATUS OF GETE TOE C'O PERADIATION ?

GEOLOGICAL CONDITIONS GENTEROTURE) DO NOT CEFLETT CONDITIONS THAT MAY OCCUR IN COMMERCIAL USE. HIGH RATES OF YENTHATION MAY BE REQUIRED.

21 MO. IN PIR. 0.0003 HOH CORROSION.

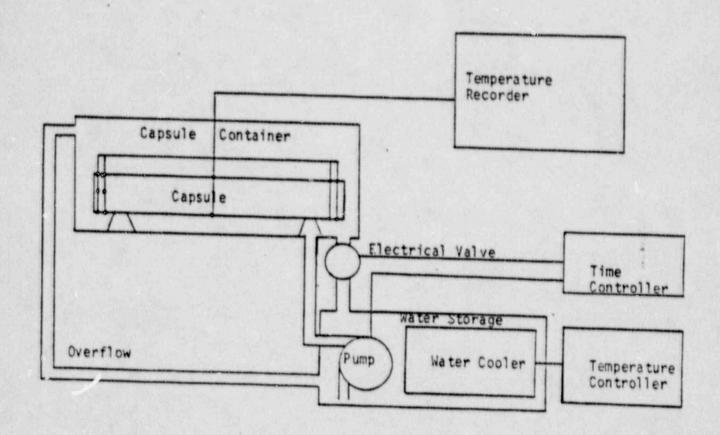
THERMAL CHOLING FOR 4400 CYCLES - 640.

EXAMWATION

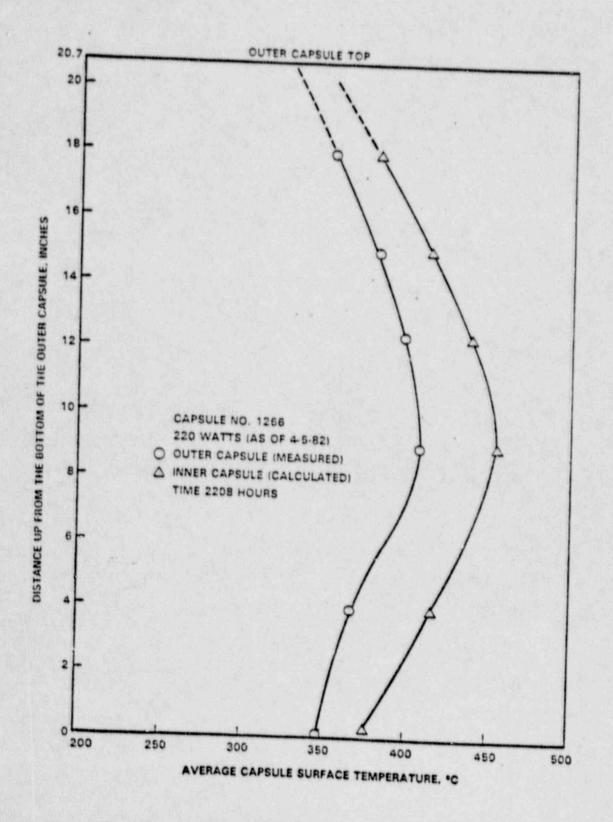
PERFORMANCE.

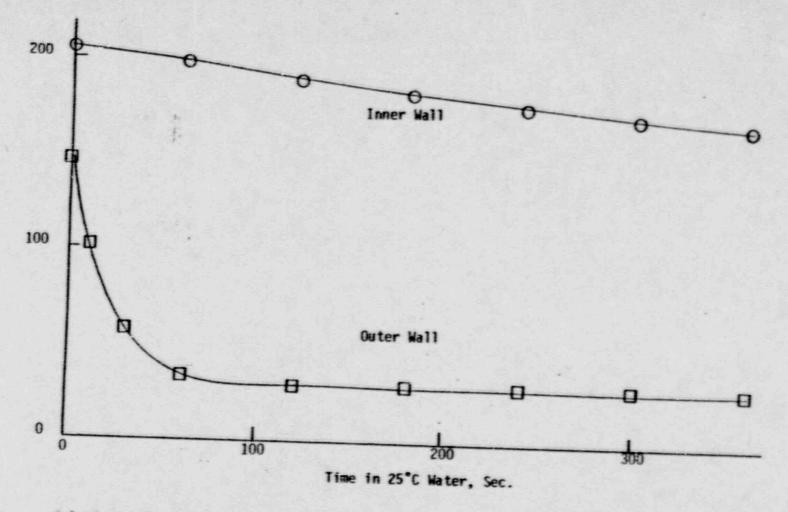
WHA JICHA WILL GIVE IB, IB, & IV IN CATER WERESPONDENCE.

THERMAL CYCLING TEST EQUIPMENT

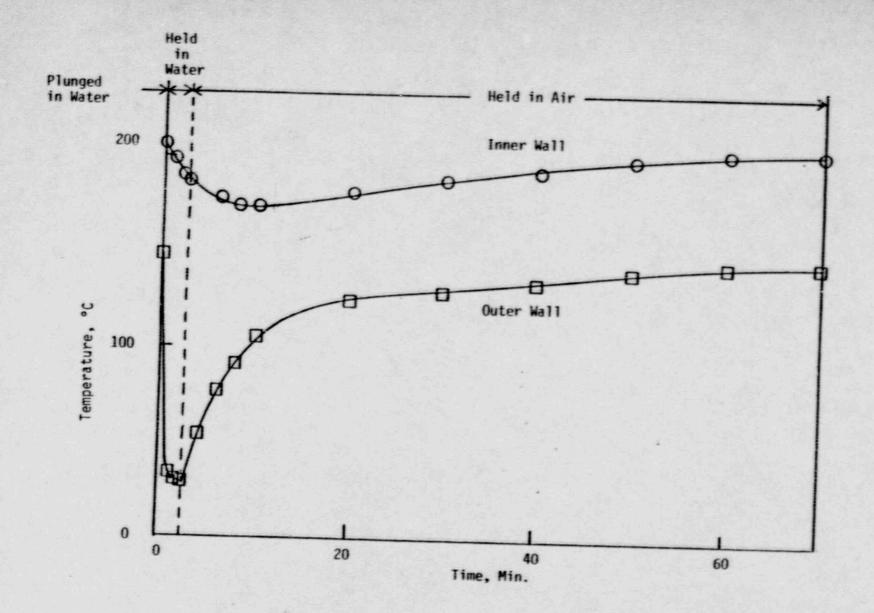


- Thermal cycling is accomplished by pumping water in and draining water out of the capsule container at a specified schedule.
- The overflow permits continuously cycling cool water from the sump to the capsule container during capsule cooling, facilitating a constant temperature bath and/or a more surbulent cooling cycle.

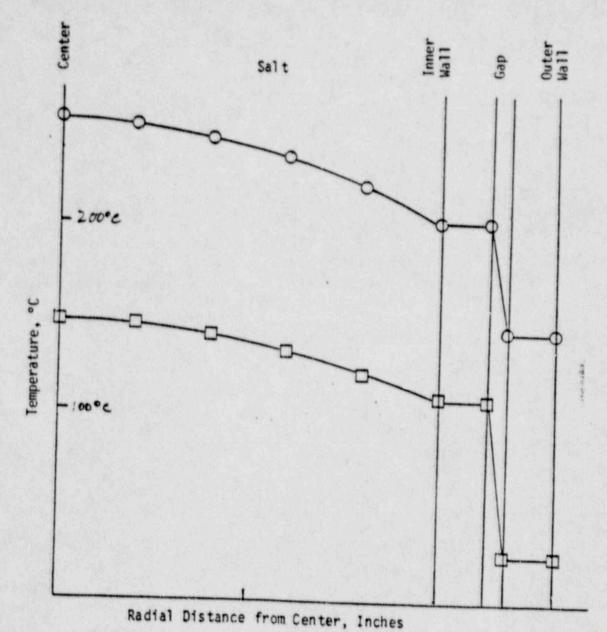




Calculated Temperature Versus Time for a 75 kCi WESF Cesium Chloride Capsule Initially at Equilibrium in 25°C Air, Then Suddenly Plunged into 25°C Water



Calculated Temperature Versus Time for a 75 kCi WESF Capsule Initially in Equilibrium in 25°C Air, Then Suddenly Plunged into 25°C Water and Held for 150 Sec, Then Held in Air Again



Calculated Temperature at Mid-length Versus Radial Distance from the Center of 75 kCi WESF Cesium Chloride Capsule at Equilibrium. O- capsule in 25°C air; — capsule in 25°C water. The inner and outer capsules are assumed to be concentric.

DOE DEMONSTRATION FACILITIES

11 SIDSS (Wet Load/Dry Operate) Intermittent Operation Since 1979 MCI Co-137 (15 MESF Cocades)

- L' Dansfortable Cesius Irradions (TRCI)....
- (Dry. Land/Dry. Operate)

 *** Usense Application to MCC Reign: FYES Operational 3rd Other FY85 250 KCI Cs-137 (4 KESF Capades)
- 3. Cesium Agricultural Commodities Pradiates (CACI) (Wet Load/Wet Operate)
 - Cicense Application to NRSC 246 Co. FYSS Operational USE Cor. FYSS 3 MCF.Cs-137 (50 WESF Capsules)

The same of the sa

- Fresh Pork Irradiator Concept Undetermined 2-3 MC C-137
- Alloquerque Senage Sandor Irradiator (Bet Lord/Dry Operate) 72-15-NG Cs-137 Status Uncertain
- 6. Related Commercial Itradiators
 Using Cs-137 for Medical Products Sterilization