PSE&G

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

March 15, 1990
HSE-90-066

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for February are being forwarded to you with the summary of changes, tests, and experiments for February 1990 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan
General Manager -
Hope Creek Operations

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Attachment

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354

UNIT Hope Creek

DATE 3/15/90

COMPLETED BY S. Loeper

TELEPHONE (609) 339-5257

MONTH February 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1014	17	1021
2	997	18	1028
3	1026	19	1024
4	997	20	1023
5	1017	21	1012
6	1034	22	1037
7	1018	23	997
8	1054	24	834
9	980	25	1021
10	1010	26	1019
11	1030	27	1007
12	1023	28	1046
13	1028	29	
14	1032	30	
15	1035	31	
16	977		

OPERATING DATA REPORT

DOCKET NO. 50-354
UNIT Hope Creek
DATE 3/2/90
COMPLETED BY S. Loeper
TELEPHONE (609) 339-5257

OPERATING STATUS

1. REPORTING PERIOD February 1990 GROSS HOURS IN REPORTING PERIOD 672

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt) 3293
   MAX. DEPEND. CAPACITY (MWe-Net) 1031
   DESIGN ELECTRICAL RATING (MWe-Net) 1067

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) None

4. REASONS FOR RESTRICTION (IF ANY)

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NO. OF HOURS REACTOR WAS CRITICAL	672	1278.2	23,039.7
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
7. HOURS GENERATOR ON LINE	672	1257.5	22,608.7
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9. GROSS THERMAL ENERGY GENERATED (MWH)	2,145,587	3,942,937	70,899,313
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	709,020	1,297,240	23,453,503
11. NET ELECTRICAL ENERGY GENERATED (MWH)	680,313	1,241,204	22,397,753
12. REACTOR SERVICE FACTOR	100	90.3	82.3
13. REACTOR AVAILABILITY FACTOR	100	90.3	82.3
14. UNIT SERVICE FACTOR	100	88.8	80.7
15. UNIT AVAILABILITY FACTOR	100	88.8	80.7
16. UNIT CAPACITY FACTOR (Using MDC)	98.2	85.0	77.6
17. UNIT CAPACITY FACTOR (Using Design MWe)	94.9	82.2	74.9
18. UNIT FORCED OUTAGE RATE	0	11.2	5.9

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION):
    None
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF START-UP:
    N/A

OPERATING DATA REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354

UNIT Hope Creek

DATE 3/15/90

COMPLETED BY S.Loeper

REPORT MONTH February, 1990 TELEPHONE (609) 339-5257

NO.	DATE	TYPE F FORCED S SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/ COMMENTS
						NONE

SUMMARY

REFUELING INFORMATION

COMPLETED BY: Chris Brennan

DOCKET NO: 50-354
UNIT NAME: Hope Creek Unit 1
DATE: 3/15/90
TELEPHONE: (609) 339-3193
EXTENSION: N/A

Month February 1990

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: 01/19/91

3. Scheduled date for restart following refueling 03/05/91

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO X

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?

   YES _____ NO X

   If no, when is it scheduled? 02/01/91

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:
   -Amendment 34 to the Hope Creek Tech Specs allows the cycle -specific operating limits to be incorporated into the CORE -OPERATING LIMITS REPORT; a submittal is therefore not required.

7. Number of Fuel Assemblies:
   A) Incore 764
   B) In Spent Fuel Storage (prior to refueling) 496
   C) In Spent Fuel Storage (after refueling) 744

8. Present licensed spent fuel storage capacity: 1290

   Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 03-05-91

HOPE CREEK GENERATING STATION
MONTHLY OPERATING SUMMARY
FEBRUARY 1990

Hope Creek entered the month of February at 95% power; the limitation was due to the 2C Drain Cooler being out of service. After consultation with General Electric, power was raised to 98% on February 6. The unit operated throughout the month without experiencing any shutdowns or reportable power reductions. On February 28, the plant completed its 49th day of continuous power operation.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS

FOR THE HOPE CREEK GENERATING STATION

FEBRUARY 1990

The following Design Change Package (DCP) has been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

The DCP did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. The DCP did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

DCP	Description of Design Change Package
4HM-0124	This DCP installed a door between the Radwaste Building Controlled Instrument Shop and the Radwaste Building Uncontrolled Instrument Shop Annex to allow the Radwaste Building Controlled Instrument Shop to be outside of the Radiological Control Area. This room was originally scheduled to be used as a hot shop, but was never used for that purpose.

The following Temporary Modification Requests (TMR's) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

The TMR's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The TMR's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Safety Evaluation	Description of Temporary Modification Request
90-029	This Safety Evaluation authorized a revision to the Temporary Modification Administrative Procedure. This revision provides administrative control, other than a TMR, for temporary floor drain plugging. Temporary floor drain plugs will have a tagging request associated with their installation. A monthly audit will be performed on plugs which will verify that the plugs are in place, properly installed and tagged. The need for continued use of the plugs will also be reviewed monthly. Precautions will be taken to ensure ECCS divisions are maintained by allowing temporary floor drain plugs to be used in only one ECCS division at a time.
90-036	This TMR rewired the control circuitry of the Reactor Core Isolation Cooling Isolation Valve Bypass Valve. Rewiring the circuitry will allow the valve to operate correctly and will allow full closure to be verified. This TMR was necessitated by a ground in the circuitry.
90-044	This TMR added a jumper to the High Oil Temperature Switch in the "A" Control Room Water Chiller. The jumper will disable the High Bearing Oil Temperature trip; but will permit the chiller to operate until replacement parts can be installed. Equipment Operators will monitor the oil temperature as a compensatory measure.