Mile Fold Ste         Like NUCL LAR REPORT (LER)         APPROVED OWN NO STROTTOR           LICENSEE EVENT REPORT (LER)         EVENS 2 6020         ESTANTO BURDES TO COMPLY WITH THE RECORD STOCEMENT OF UNDER THE REPORTS TO COMPLY WITH THE RECORD STOCEMENT OF UNDER THE RECORD STOC					_															
LICENSEE EVENT REPORT (LER)         EXEMPLE         EXE		RM 366	:							U.S. NU	ICLEAR RE	GULATOR	V COMMISS	SION					•	
LICENSEE EVENT REPORT (LER)         Decket water to the RECORD ENDERSE BURGER STATE TO THE RECORD ENDERSE BURGER STATE TO THE RECORD ENDERSE BURGER STATE TO THE RECORD AND THE APPENDENCE STATEMENT AND RECORD AND AND AND AND THE APPENDENCE STATEMENT AND RECORD AND AND AND AND AND AND AND AND AND AN															ESTIMATE	D BURDEN PER	R RESP	ONSE TO CO	MPLY WT	-
REGULATORY COMMISSION WASHINGTON, DC 2006. AND TO DE GARAGEMENT AND DE COMMISSION WASHINGTON, DC 2006. TO DE GARAGEMENT AND DE COMMISSION WASHINGTON, DC 2006. TO DE GARAGEMENT AND DE COMMISSION DATE:           FACILITY NAME (1) DOCKTY WASHINGTON, DC 2006. TO DOCKTY WASHINGTON, DC 2007. TO POWER SUPPLY FVENT DATE (8) DOCKTY WASHINGTON, DC 2007. TO FVENT DATE (8) DOCKTY WASHINGTON, DC 2007. TO FVENT DATE (8) DOCKTY WASHINGTON, DC 2007. TO FVENT DATE (8) FVENT DATE (8	P. A.				LIC	ENS	SEE E	VENT	REP	ORT	LER)				COMMENT	S REGARDING I	BURDEN	N ESTIMATE '	TO THE RE	ECORDS
OF MARADEMENT AND BUCKTY MASHINGTON D.2 2000.           FACILITY NAME (II)         Shoreham Nuclear Power Station Unit 1         DOCKET NUMBER (I)         FACILITY NAME (II)           THE RACUL Isolation Due to Inadequate Procedure When Performing Calibration of 24 VDC           Power Station Unit 1         OTHER #ACLUTES INVOLVED IN           NOVER Station Due to Inadequate Procedure When Performing Calibration of 24 VDC           Power Station Due to Inadequate Procedure When Performing Calibration of 24 VDC           Power Station Due to Inadequate Procedure When Performing Calibration of 24 VDC           Power Station Due to Inadequate Procedure When Performing Calibration of 24 VDC           NAME FOR USE Station Due to Inadequate Procedure When Performing Calibration of 24 VDC           OTHER #ACLUTES NUMBER (III)           OTHER #ACLUTES NUMBER (III)           OTHER #ACLUTES NUMBER (III)           OTHER #ACLUTES NUMBER (IIII)           OTHER #ACLUTES NUMBER (IIII)           OTHER #ACLUTES NUMBER (IIIII)           OTHER #ACLUTES NUMBER (IIIII)           OTHER #ACLUTES NUMBER (IIIIII)           OTHER #ACLUTES NUMBER (IIIIII)           OTHER #ACLUTES NUMBER (IIIIIIIIIIIIIIIIIIIIIIIIIIIIIIIIIIII															REGULAT	DRY COMMISSI	ON WAS	SHINGTON, D	C 20555	AND TO
PARCELITY RANK (1)         Shoreham Nuclear Power Station Unit 1         0   5   0   0   0   3   2   2   0   0   4   3   2   0   0   0   4   3   2   0   0   0   4   3   2   0   0   0   4   3   2   0   0   0   1   1   1   0   5   0   0   0   1   1   1   0   5   0   0   0   1   1   1   0   0   0   1   1	100														OF MANAL	SEMENT AND B	UDGET.	WASHINGTO	N. DC 205	03.
Title (#)         RWCU Isolation Due to Inadequate Procedure When Performing Calibration of 24 VDC           Power Supply         EVENT Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           MONTH         Date (#)         EEPOND Date (#)         Other Faculty is involved #)           Old Earlie         EEPOND Date (#)         Date (#)         Date (#)         Other Faculty is involved #)           Old Earlie         EEONT Date (#)         EEONTH EED FURGURANT TO THE FACULTY EED FURGURANT ED	FACILIT	Y NAME	(1)							a a a d a		+ 1			D	OCKET NUMBE	R (2)			
POWER         SUDDLY           EVENT DATE (0)         LER NUMBER (0)         REPORT DATE (7)         OTHER FACILITIES INVOLVED (0)           MONTH         DAY         YEAR         FACILITY NAMES         DOCKET NUMBER (0)           MONTH         DAY         YEAR         FACILITY NAMES         DOCKET NUMBER (0)           MONTH         DAY         YEAR         FACILITY NAMES         DOCKET NUMBER (0)           0         1         2         9         9         0         0         1         0									_							makes makes	- Annual	and the second se		014
EVENT DATE (B)         LEP SUBJECT         ALPONT (ALT (F)         OTHER FACILITIES INVOLVED (B)           MONTH         DAY         VEAR         95 (UNBER (ALT (ALT (ALT (ALT (ALT (ALT (ALT (ALT	TITLE					n Du	le to	Inade	equa	ite Pi	rocedu	ire Wh	en Per	rfor	ming (	Calibrat	ion	of 24	VDC	
NONTH         DAV         VEAN         NUMBER         NUMARITION         NUMARITION         NUMAR	Ev			T		and the second		A DESCRIPTION OF THE PARTY OF		RE	PORT DAT	E (7)			OTHER I	ACILITIES INV	and the second second	The second se		
011         219         9         0         0         5         0         0         1         1 <td>MONTH</td> <td>VAG</td> <td>YE</td> <td>LR I</td> <td>VEAR</td> <td>5</td> <td></td> <td>AL N</td> <td></td> <td>MONTH</td> <td>DAY</td> <td>YEAR</td> <td></td> <td>**</td> <td>CILITY NAM</td> <td>65</td> <td>DOC</td> <td>KET NUMBER</td> <td>1(5)</td> <td></td>	MONTH	VAG	YE	LR I	VEAR	5		AL N		MONTH	DAY	YEAR		**	CILITY NAM	65	DOC	KET NUMBER	1(5)	
OPERATING MODE (B)         THE REPORT IS BURNITED FURBURNT TO THE REQUIREMENT OF TO CRE E (Check one or more of the following) (T1)           POWER EVEL         20 402(b)         20 402(b)         20 402(c)																	0	51010	101	11
OPERATING MODE (B)         THE REPORT IE BUBMITTED FURBURNT TO THE REDUREMENT OF TO CRE E (Check one or more of the following) (T1)           POWER LEVEL         26 402(b)         20 405(c)         20 405(c	1.1	1 .	1.			_		-	11	1							1.			
OPERATING         V         20 402(b)         20 406(c)         X         50.73(c)(2)((c)         73.71(c)           POWER         20 406(c)(11(0)         50.36(c)(1)         50.36(c)(1)         50.73(c)(2)((c)         73.71(c)           POWER         20 406(c)(11(0)         50.36(c)(1)         50.36(c)(1)         50.73(c)(2)(c)         50.73(c)(2)(c)         73.71(c)           POWER         20 406(c)(11(0)         50.36(c)(1)         50.73(c)(2)(c)         50.73(c)(2)(c)         73.71(c)           POWER         20 406(c)(11(0)         50.36(c)(2)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         73.71(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         73.71(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         73.71(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)(c)         50.73(c)(2)(c)         50.73(c)(2)(c)           20 406(c)(11(0)         50.73(c)(2)(c)(c)         50.73(c)(2)(c	01	29	9			10	101	3 10	010	11								51010	101	11
POWLE     26 406(in1110)     26 406(in1110)     26 38(in12)     26 3			0	1	-		S SUBMIT	TED PURS	BUANT	·····		INTE OF 10				( the following)	11)			
ENVEL       0.1010       20.406(e1016(e))       60.38(e12)       60.38(e12)       60.73(e12)(e001       <				*					-	-			×	1			H			
No.     D <thd< th="">     D<!--</td--><td>LEVI</td><td>1</td><td></td><td></td><td></td><td></td><td></td><td></td><td>-</td><td>-</td><td></td><td></td><td>-</td><td></td><td></td><td></td><td>-</td><td></td><td></td><td>ATTACT</td></thd<>	LEVI	1							-	-			-				-			ATTACT
20 406(a)(11)(w)     B0.73(a)(2)(w)     B0.73(a)(2)(w)       20 406(a)(11)(w)     B0.73(a)(2)(w)     B0.73(a)(2)(w)       00 73(a)(2)(w)     A0.0000       00 73(a)(0)(w)     A0.0000	001	10	101	0					-	-			-				H	below and in		
20.409 (a)(11(k))     50.73 (a)(2)(4))     80.73 (a)(2)(4)       LICENSEE CONTACT FOR THIS LER (12)       TELEPHONE NUMBER       AAME       George D. Schnaars, Operational Compliance Engineer       COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REFORT (13)       COMPONENT MANUFAC REFORTABLE       COMPONENT MANUFAC REFORTABLE       TURER       COMPONENT MANUFAC REFORTABLE       COMPONENT MANUFAC REFORTABLE       COMPONENT MANUFAC REFORTABLE       COMPONENT MANUFAC REFORTABLE       CAUSE SYSTEM       COMPONENT MANUFAC REFORTABLE       TURER       BUPPLEMENTAL REPORT EXPECTED (14)       EXPECTED SUEMISSION DATE)       NO				H					-	-			-							
LICENSEE CONTACT FOR THIS LER (12)  NAME  George D. Schmaars, Operational Compliance Engineer  COMPLETE DNE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REFORT (13)  CAUSE SYSTEM COMPONENT MANUFAC REPORTABLE TO NFRDS  CAUSE SYSTEM COMPONENT MANUFAC REPORTABLE SUBMISSION DATE NUMER NO				H					-	-			-	1						
TELEPHONE NUMBER       TELEPHONE NUMBER       George D. Schnaars, Operational Compliance Engineer       COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REFORT 1131       CAUSE SYSTEM COMPONENT       MANUFAC REPORTABLE       TURER       OMPONENT       MANUFAC REPORTABLE       CAUSE SYSTEM COMPONENT       MANUFAC TURER       TURER       BUPPLEMENTAL REPORT EXPECTED (14)       EXPECTED SUBMISSION DATE)       ND					1				-					1			-			
George D. Schnaars, Operational Compliance Engineer  COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DEBCRIBED IN THIS REPORT (13)  CAUSE SYSTEM COMPONENT MANUFAC REPORTABLE TO NURRE CAUSE SYSTEM COMPONENT MANUFAC REPORTABLE SUPPLEMENTAL REPORT EXPECTED (14)  EXPECTED SUPPLEMENTAL REPORT EXPECTED (14)  VES (III vel. complete EXPECTED SUBMISSION DATE)  X NO	NAME					-		-								1	TELE	PHONE NUM	BER	
CAUBE EVETEM COMPONENT MANUFAC REPORTABLE CAUBE SYSTEM COMPONENT MANUFAC REPORTABLE TO NORDS																AREA CODI	T			
CAUBE EYSTEM COMPONENT MANUFAC REPORTABLE TURER TO NORDS CAUBE SYSTEM COMPONENT MANUFAC REPORTABLE TURER TO NORDS SUPPLEMENTAL REPORT EXPECTED (14) YES (III Val. complete EXPECTED SUBMISSION DATE) X ND	Geo	rge	D. S	chn	aars	, 0	perat	cional	1 Ca	mplia	ince E	ngine	er			511 6	91	2191-	18 13	10 10
CAUSE SYSTEM COMPONENT TURER TO NPROS							COMPLE	TE ONE LI	INE FOR	R EACH C	OMPONENT	FAILURE	DESCRIBE	ED IN 1	THIS REFOR	T (13)				
ND     ND		Levere				-	NUFAC					CAUSE	EVETEN	00	MPONENT	MANUFAC	-	PORTABLE		
YES (II VOL COMPLETE EXPECTED SUBMISSION DATE) X NO	CAUGE					-	URER	TON	VPRDS							TURER	-	TO NPRDS		
VES (III VAL COMPLETE EXPECTED SUBMISSION DATE) X NO					10.0	11.1						e Bata		100						
YES (II VOL COMPLETE EXPECTED SUBMISSION DATE) X NO		11	-		1	1				+			+							
VES (III VAL COMPLETE EXPECTED SUBMISSION DATE) X NO	1.1.1					135						1.4	1.1	1						
VES (III VAL COMPLETE EXPECTED SUBMISSION DATE) X NO		11			_					i				1				Lucerte	Torr	Typen
VES (II VOL COMPLETE EXPECTED SUBMISSION DATE) X NO DATE (15)							SUPPLE	MENTAL	REPOR	EXPECT	ED (14)							MONTH	- DAT	TEAN
					-	SURM	IRRIDA D.			+	NO					DATE	(15)		1 .	1.
	-			-	-				and the	12	<u></u>					d				
On January 29, 1990 at 1125 hours, an unplanned actuation of the	1 1	Engi	nee	rec	d Se	af et	ty F	aatu	re i	Prim	ary C	ionta	i.rim@i	nt	(sola	tion 5	ysti	em		
In January 29, 1990 at 1125 hours, an unplanned actuation of the Engineered Safety Feature Primary Containment Isolation System		DCCL	(r r e	d.	Th	115	EVE	nt or	CCU	rred	duri	ng t	he ca	i la	brati	on of i	a 24	4 VDC		
	1	OWE	er E	upp	yic	in	the	Rear	ctor	r Wa	ter C	lean	UD (F	RWC	U) Le	akage 1	Detr	ection		
Engineered Safety Faature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC																				
Engineered Safety Faature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection																				
Engineered Safety Faature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection circuitry. The Technician lifted leads to deenergize the power																				
Engineered Safety Faature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection																			U	
Engineered Safety Feature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection circuitry. The Technician lifted leads to deenergize the power supply which also deenergized the RWCU Leak Detection Ambient																				
Engineered Safety Fzature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection circuitry. The Technician lifted leads to deenergize the power supply which also deenergized the RWCU Leak Detection Ambient Temperature Switches 1633*TS-071 B, D and F. When the leads were	1	stop	ped	ar	nd t	the	150	latic	on i	マの時に	reset	. T	he RI	WCU	syst	en was	ree	stored		
Engineered Safety Feature Primary Containment Isolation System occurred. This event occurred during the calibration of a 24 VDC power supply in the Reactor Water Cleanup (RWCU) Leakage Detection circuitry. The Technician lifted leads to deenergize the power supply which also deenergized the RWCU Leak Detection Ambient Temperature Switches 1633*TS-071 B, D and F. When the leads were reconnected a high temperature trip occurred which caused the RWCU containment isolation valve 1633*MDV-034 to close. The work was stopped and the isolation was reset. The RWCU system was restored	1000	to i	ts	nor	mail	1:	Deu	p and	d pi	lant	mana	geme	nt wi	截集	infor	med.	Than	s even	t	

is reportable per 10CFR50.72(b)(2)(ii) and the NRC was notified at 1414 hours. The cause of this event was inadequate procedural guidance. Corrective actions include modifying the Preventive Maintenance Worksheet to specify which leads to remove to deenergize the power supply and also adding a precaution to the power supply calibration procedure.

\* Reactor Defueled

9003090048 900223 PDR ADOCK 05000322

LICENSEE EVENT RE		APPROVED DME NO. 3150-0104 EXPIRES 4/30/82 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH TI INFORMATION COLLECTION REDUEST SOCI HRS. FORWA COMMENTS REGARDING BURDEN ESTIMATE TO THE RECOP AND REFORTS MANAGEMENT BRANCH (F-330). US NUCLE REGULATORY COMMISSION WASHINGTON DC 20555. AND								
		THE FAPERWORK REDUCTION PR	OJECT (3150-0104), OFFICE							
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)							
Shoreham Nuclear Power Station Uni	+ 1	NUMBER								
TEXT (If more space is required, use additional NRC Form 306A 's/ (17)		<u> 010 -010 3 -01</u>	0 10 12 1-10 14							
PLANT_AND_SYSTEM_IDENTIFI	CATION									
General Electric - Boilin	g Water Reactor									
Energy Industry Identification System (EIIS) codes are identified in the text as [xx].										
IDENTIFICATION OF THE EVE	NT									
Unplanned isolation of the occurred during performan			RGK.							
Event Date: 1/29/90										
Report Date: 2/23/90										
CONDITIONS PRIOR TO THE E	YENT									
Reactor Defueled - All fu	el assemblies stored :	in the Spent Fue)	Popl							
Made Switch - Shutdown										
RPV Pressure = 0 psig	RPV Temperature	= 88 Degrees F								
POWER LEVEL - 0										
DESCRIPTION_OF_THE_EVENT										
On January 29, 1990 preve E/S-800B-4001 was being p VDC power supply in the R Detection circuitry. It 46.007.02, 24 VDC Power S lifted 2 leads to deenerg one of these leads also d Leak Detection Ambient Te of the power supply calib reconnected the lifted le temperature switches to r trip and closing of the R 1633*MOV-034. This unpla Feature Frimary Containme 10CFR50.72(b)(2)(ii). Th and the RWCU System was r management was notified o	erformed. This is a eactor Water Cleanup is performed per Stat upply Functional Test ize and unload the po eenergized 1633*T5-07 mperature Switches). ration was completed ads at 1125 hours. The eenergize, resulting WCU containment isolat nned actuation of the nt Isolation System ( e work was stopped, the estored to its normal	calibration of th (RWCU) Leakage ion Procedure . The Technician wer supply; 1 B. D. and F (RW The no-load port satisfactorily. his caused the th in a high tempera- tion valve Engineered Safet JMJ is reportable he isolation was lineup. Plant	NCU tion He aree ature ty e per reset							

NRC Form 386A (6-89)

hours.

ILICENSEE EVENT	COMM AND REGL THE	REPO	BUR ION C REG RTS N RY C	E DEN PI COLLECT ARDING CANAGE OMMISS K REDU	APIRES ER RES TION & BURD MENT HON, W	AB NO 311 S 4/30/92 SPONRE AEQUEST DEN ESTIM BRANCH VASHINGT N PROJEC T, WASHI	TO 0 60.0 MATE (P-6) TON, DT (3	0 MPL 0 MPIS TO TH 30). U. DC 200	5. FOI	CORD	ID DS HO	
ACILITY NAME (1)	DOCKET NUMBER (2)		LE	R NU	-	i.)		T	**	GE (		
		YEAR			INTIAL		REVISION			П		
Shoreham Nuclear Power Station	0   5   0   0   3   2   2	910	_	0	0 3	-	010	0	13	OF	0	4
TEXT IN more space is required, use additional NRC Form 3864 's) (17)			12			in the second	and the second s		-			

## CAUSE OF THE EVENT

This event was caused by inadequate procedural guidance. The Preventive Maintenance Scheduled Activity Worksheet (PM SAWS) contained insufficient instructions for preventing a RWCU isolation by requiring three circuit cards be removed prior to deenergizing the power supply. Also the PM SAWS did not specifically identify which leads had to be removed to deenergize the 24 VDC power supply.

When the two leads were lifted, the 24 VDC power supply and temperature switches 1G33\*T5-071 B, D and F were deenergized. The design of these temperature switches is such that when they are initially energized they go full scale high and then settle back to whatever is input from their associated temperature elements. When the leads were reconnected they reenergized the temperature switches which caused a high temperature trip and initiated a close signal to the containment isolation valve 1G33\*MOV-034.

## ANALYSIS OF THE EVENT

There was no safety significance to this event. The plant is shutdown and has been defueled since August of 1989. The operators reset the RWCU isolation and restored the system to its normal inneup.

## CORRECTIVE ACTIONS

- The FM SAWS for the 24 VDC power supply will be revised so that it states which leads are to be removed to deenergize only the power supply.
- SP 46.007.02, 24 VDC Power Supply Functional Test, will have a precaution added that requires the Technician to contact a supervisor if he is uncertain of which leads to be removed.
- 3. This event will be reviewed by all I&C personnel.
- This event will be documented on the Equipment History Card for the 24 VDC power supply to inform other Technicians about the potential for causing a RWCU isolation.

LICENSEE EVENT REPORT TEXT CONTINUATION		APPROVED DME NO. 3160-0104 EXPIRES 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWAND COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH IPS.57. U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 2056, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 2050.								
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER IS	PAGE (3)							
Shoreham Nuclear Power Station Unit 1		YEAR SEQUENTIAL REVISION								
	0   5   0   0   3   2   2	910 -01013 -010	014 050 14							
TEXT (# more space is required, use additional NAC Form 2004/2/(17)										
ADDIIIONAL_INEOBMAIION										

a. Manufacturer and model number of failed component (s)

N/A

b. LER numbers of previous similar events

86-035