



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION I  
631 PARK AVENUE  
KING OF PRUSSIA, PENNSYLVANIA 19406

AUG 17 1981

Docket Nos. 50-354  
50-355

MEMORANDUM FOR: *Blackwood*  
E. Blackwood, Chief, Reactor Projects Section, DRRRI, IE  
THRU: R. R. Keimig, Chief, Projects Branch #2, Division of Resident  
and Project Inspection  
FROM: E. G. Greenman, Chief, Reactor Projects Section 2A, DRPI  
SUBJECT: REACTOR PRESSURE VESSEL AND OTHER COMPONENTS NOT MANUFACTURED  
IN ACCORDANCE WITH EFFECTIVE ASME III CODE REQUIREMENTS  
(AITS: F01004890)

The attached letter (PSE&G to NRR dated January 30, 1979) details differences between required codes listed in 10 CFR 50.55a subsections (c) through (f) and the codes used to manufacture the Hope Creek 1 and 2 reactor pressure vessels as well as other reactor system components.

Based on the expected issuance of a construction permit in 1971, the licensee's engineering design and component fabrication were accomplished using codes in effect at that time. The construction permit was issued November 4, 1974.

Region I requests that evaluation status and resolution of the licensee's request for code exemption be provided.

*L. E. Briggs for*  
E. G. Greenman, Chief  
Reactor Projects Section 2A, Division  
of Resident and Project Inspection

Enclosure: As Stated

CONTACT: L. Briggs  
488-1237

January 30, 1979

Director of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

P S E & G HOPE CREEK SITE QAD	
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Gentlemen:

HOPE CREEK GENERATING STATION  
DOCKET NOS. 50-354 AND 50-355  
CODES AND STANDARDS FOR REACTOR  
COOLANT PRESSURE BOUNDARY EQUIPMENT

Pursuant to Section 50.55a(a) (2) (i) and (ii) of 10 CFR Part 50, Public Service Electric and Gas Company hereby requests authorization to use certain components in its Hope Creek Generating Station, Units 1 and 2 which were designed to codes which differ from those specified in Subsections (c) through (f) of 10 CFR 50.55a. The specific components for which authorization is requested are set forth in the table attached hereto.

On February 27, 1970, Public Service Electric and Gas Company filed its construction permit application for Hope Creek, Units 1 and 2. Engineering and Construction schedules at that time were based on expected construction permit issuance in 1971. Initial engineering design and procurement activity was based on this forecast. The design, fabrication, and testing of components purchased in this period were based on recognized codes and standards in effect at the time. Since the construction permits were not issued until November 1974, some of the codes used differ from those now listed in 10 CFR 50.55a Subsections (c) through (f). We believe that the costs associated with replacing the components in the table solely for the purpose of changing the design codes would represent a severe hardship without a compensating increase in quality or safety. Furthermore the codes to which the subject equipment and components were designed assure the achievement of an acceptable level of quality and safety.

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In view of the above, Public Service Electric and Gas Company respectfully requests an authorization from the Commission in accordance with Section 50.55a(a) (2) (i) and (ii) of 10 CFR Part 50 to use the codes referenced for the components listed in the attachment to this letter.

Very truly yours,

R. L. Mittl  
General Manager - Licensing and Environment  
Engineering and Construction Department

JCR:jd

HOPE CREEK GENERATING STATION  
 UNITS NO. 1 & 2  
 DOCKET NOS: 50-354 AND 50-355  
 REACTOR COOLANT PRESSURE BOUNDARY  
 EQUIPMENT CODE STATUS

<u>Equipment</u>	<u>Purchase Order Date</u>	<u>Code Equipment Was Built to:</u>
Reactor Pressure Vessel-Unit One	May 7, 1970	ASME III, 1968 Edition with Winter 1969 Addenda 1968
Reactor Pressure Vessel-Unit Two	April 30, 1971	ASME III, 1968 Edition with Summer 1970 Addenda 1970
Control Rod Drive Housings	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Control Rod Drive	August 18, 1971	ASME III, 1968 Edition with Winter 1970 Addendum
Power Range Monitor Incore Housing	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Jet Pump Instrumentation Penetration	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Main Steamline safety relief valves	January 28, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Main Steamline isolation valves	October 8, 1969	Nuclear Pump and Valve Code, 1968 Edition
Main Steamline pipe	January 27, 1972	ASME III, 1971 Edition with Summer 1971 Addenda
Main Steamline flow elements	February 5, 1973	ASME III, 1971 Edition with Summer 1972 Addenda
Reactor recirculation system pump	May 7, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Reactor recirculation system shut off valves	February 23, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Reactor recirculation system, by-pass valve	December 23, 1971	ASME III, 1971 Edition
Reactor recirculation system, pipe	February 25, 1971	ASME III, 1968 Edition with Summer 1970 Addenda