LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT
3.6.E Jet Pumps	4.6.E Jet Pumps
3.6.F Jet Pump Flow Mismatch	b. The indicated value of core flow rate varies from the value derived from loop flow measurements by more than 10%. c. The diffuser to lower plenum differential pressure reading on an individual jet pump varies from the mean of all jet pump differential pressures by more than 10%. 3. Whenever there is recirculation flow with the reactor in the Startup or Run Mode and one recirculation pump is operating with the equalizer valve closed, the diffuser to lower plenum differential pressure shall be checked daily and the differential pressure of an individual jet pump in a loop shall not vary from the mean of all jet pump differential pressures in that loop by more than 10%.
1. The reactor shall not be operated with one recirculation loop out of service for more than 24 hours. With the reactor operating, if one recirculation loop is out of service, the plant shall be placed in a hot shutdown condition within 24 hours unless the loop is sooner returned to service.	F. Jet Pump Flow Mismatch 1. Recirculation pump speeds shall be checked and logged at least once per day.
2. Following one pump operation, the discharge valve of the low speed pump may not be opened unless the speed of the faster pump is less than 50% of its rated speed.	
3. Steady state operation with both recirculation pumps out of service for up to 12 hrs is permitted. During such interval restart of the recirculation pumps is permitted, provided the loop discharge temperature is within 75°F of the saturation temperature of the reactor vessel water as determined by dome pressure. The total elapsed time in natural circulation and one pump operation must be no greater than 24 hrs.	G. Structural Integrity 1. Inservice inspection of ASME Code Class 1, Class 2, and Class 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
G. Structural Integrity 1. The structural integrity of the primary system shall be	

LIMITING CONDITIONS FOR OPERATION

3.6.G Structural Integrity

maintained at the level required by the original acceptance standards throughout the life of the plant. The reactor shall be maintained in a cold shutdown condition until each indication of a defect has been investigated and evaluated.

SURVEILLANCE REQUIREMENTS

4.6.G Structural Integrity

4. Additional inspections shall be performed on certain circumferential pipe welds as listed to provide additional protection against pipe whip, which could damage auxiliary and control systems.

Feedwater	-	GFW-9, KFW-13 GFW-12, GFW-26, KFW-31, GFW-29, KFW-39, GFW-15, KFW-38, and GFW-32
Main steam	-	GMS-6, KMS-24, GMS-32, KMS-104 GMS-15, and GMS-24
RHR	-	DSRHR-4, DSRHR-7, DSRHR-6
Core Spray	-	DSCS-12, DSCS-11, DSCS-5, and DSCS-4

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.6.G Structural Integrity	4.6.G Structural Integrity
	Reactor Cleanup - DSRWC-4, DSRWC-3, DSRWC-6, and DSRWC-5
	HPCI - THPCI-70 THPCI-70A THPCI-71, and THPCI-72

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ENCLOSURE 2

JUSTIFICATION AND SAFETY ANALYSIS
LICENSE AMENDMENT REQUEST TVA BFNP TS 169
BROWNS FERRY NUCLEAR PLANT UNITS 1 AND 2
(DOCKET NOS. 50-259, -260)

This change to the technical specifications reflects the TVA Inservice Inspection program as it presently exists and as mandated by NRC in 10 CFR 50.55a(g). It does not adversely affect operation, safety margins, accident analysis, or overall plant safety.