

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE November 13, 1981

COMPLETED BY R. W. Short

TELEPHONE (402)536-4543

MONTH October, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8111230541 811113  
PDR ADOCK 05000285  
R PDR

OPERATING DATA REPORT

DOCKET NO. 50-285  
 DATE November 13, 1981  
 COMPLETED BY R. W. Short  
 TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: October, 1981
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:  
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A  
 10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>745.0</u>	<u>7,296.0</u>	<u>71,017.0</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>6,130.6</u>	<u>55,908.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>6,080.3</u>	<u>54,840.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>6,728,000.2</u>	<u>66,446,240.5</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.0</u>	<u>2,200,498.0</u>	<u>21,988,769.6</u>
18. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>2,069,985.6</u>	<u>20,768,168.5</u>
19. Unit Service Factor	<u>0.0</u>	<u>83.3</u>	<u>77.2</u>
20. Unit Availability Factor	<u>0.0</u>	<u>83.3</u>	<u>77.2</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>59.1</u>	<u>63.9</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>59.1</u>	<u>63.5</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>2.9</u>	<u>3.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: November 29, 1981

26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY	<u>      </u>	<u>      </u>
INITIAL ELECTRICITY	<u>      </u>	<u>      </u>
COMMERCIAL OPERATION	<u>      </u>	<u>      </u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1981

DOCKET NO. 50-285  
 UNIT NAME Fort Calhoun #1  
 DATE November 13, 1981  
 COMPLETED BY R. W. Short  
 TELEPHONE (402)536-4543

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-06	810918	S	1034.1	C	1	N/A	N/A	N/A	Reactor shutdown for annual refueling and maintenance outage.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

Refueling Information  
Fort Calhoun - Unit No. 1

Report for the month ending October 1981 .

1. Scheduled date for next refueling shutdown. December 1, 1982
2. Scheduled date for restart following refueling. March 1, 1983
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
Yes
  - a. If answer is yes, what, in general, will these be?  
A Technical Specification change.
  - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.  
\_\_\_\_\_
  - c. If no such review has taken place, when is it scheduled?  
\_\_\_\_\_
4. Scheduled date(s) for submitting proposed licensing action and support information. September 1, 1982
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>237</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by JR Gasser Date November 3, 1981

OMAHA PUBLIC POWER DISTRICT  
Fort Calhoun Station Unit No. 1

October 1981  
Monthly Operations Report

I. OPERATIONS SUMMARY

The refueling outage continued during the month. Core loading for Cycle Seven was completed.

Operations supported all phases of refueling operations as well as other special maintenance activities.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
LER 81-009	During cold shutdown, containment cooling fan/coil VA-7C/VA-8A was placed in service to provide containment air cooling. When pressurized, a Component Cooling Water (Auxiliary Coolant) leak developed in coil VA-8A when a coil end plug blew out. The loss of the cooling coil end plug caused a spill of 100 to 200 gallons of Auxiliary Coolant inside the Containment. Once the source of the leak was determined, coil VA-8A was isolated to terminate the leak.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incidents</u>	<u>Deficiency</u>
OI-1383	ST-ERM-1, RM-1 Unable to get well sample as specified in Surveillance Test.
OI-1387	ST-ESF-3, F.2 Containment Pressure Sensors A/742-1, A/742-2, B/742-1, B/742-2, C/742-2, D/742-1, D/742-2 out of tolerance.

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (Continued)

<u>Operations Incidents</u>		<u>Deficiency</u>
OI-1388	ST-RPS-8, F.2	High Containment Pressure Channel A/PC-765, C/PC-765, D/PC-765 out of specification.
OI-1407	ST-ESF-8	LT-2904Y, LT-2924Y, CT-2944Y and LT-2964Y out of tolerance.

D. CHANGES, TESTS AND EXPERIEMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-GRIND-1	The work done under this procedure does not constitute an unreviewed safety question as defined by 10CFR50.59 as it only involved the preparation of a piping surface for inspection required by Technical Specification 3.3 in accordance with the ASME Boiler and Pressure Vessel Code.
SP-ISI-SURFPREP-1	The work done under this procedure does not constitute an unreviewed safety question as defined by 10CFR50.59 as it only involved the preparation of a piping surface for inspection required by Technical Specification 3.3 in accordance with the ASME Boiler and Pressure Vessel Code.
SP-ISI-SURCHPREP-1	The work done under this procedure does not constitute an unreviewed safety question as defined by 10CFR50.59 as it only involved the preparation of a piping surface for inspection required by Technical Specification 3.3 in accordance with the ASME Boiler and Pressure Vessel Code.
SP-FAUD-1	Fuel Assembly Uplift Core Detection.  An unreviewed safety question as defined in 10CFR50.59 did not exist as this procedure only involves evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

Annual leak rate tests are in progress.  
A consolidated report will be made at the end of testing.

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

Training for October included general employee training radiation protection training to support plant refueling modifications.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE

II. MAINTENANCE (Significant Safety Related)

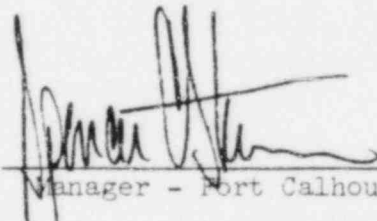
M.O. #	Date	Description	Corrective Action
10482	9-30-81	VOPT Meter RPS "A" Channel malfunctioning.	Replaced meter.
9422	10-2-81	Remove upper guide structure lift rig to upper level for inspection.	Relocated upper guide structure to southwest side of containment.
12273	10-17-81	Perform polar crane check.	Polar crane checked per procedure MP-HE-1-A.
9313	10-19-81	Replace spent fuel pool canal gate.	Spent fuel pool canal gate replaced per procedure FH-16-2.
9482	10-17-81	Change out Raw Water strainer AC-12A.	AC-12A was changed out per procedure MP-AC-12.
11691	9-23-81	Short in C Channel RRS High Voltage Cable to Wide Range Drainer.	Repaired connector.
12187	10-12-81	HCV-506A failed shut.	Replaced blown fuse.
10133	9-26-81	Replace RC-2B sec. hand hole covers.	Replaced gaskets and covers.
10811	10-20-81	Replace pressurizer manway per MP-RC-4-3.	Completed per procedure.
9366	9-26-81	Replace secondary hand hole covers per MP-RC-2-5-13.	Installed new gaskets and covers.
12291	10-25-81	HCV-2504B stuck in open position.	Valve OK-faulty limit switch, were blocking contact. Repaired - OK.
12040	10-13-81	FI-330 has tubing leak.	Tightened fitting.
12245	10-17-81	Leak in cooling coil VA-8A.	Retapped tubing and reinserted plug into tubing. Hydroed system.
12322	10-23-81	Clean Reactor Vessel Head Flange and install O-Rings.	Cleaned Reactor Vessel Flange and installed O-Rings per MP-RC-6
12215	10-13-81	Manual Operator on HCV-2883A would not disengage from manual operation.	Operator disengaged.



II. MAINTENANCE (Significant Safety Related)

M.O. #	Date	Description	Corrective Action
9409	10-24-81	Replace hold down ring.	Hold down ring replaced per procedure MP-RC-7-3-B.
11941	10-24-81	Remove Steam Generator "A" Secondary manway covers.	Manway covers opened per procedure MP-RC-2-2-A.
11575	10-21-81	Remove Alignment Pins.	Alignment pins removed per procedure MP-RC-6-5-A.
11943	10-24-81	Remove "B" Steam Generator secondary manway covers.	Secondary manway covers removed per procedure MP-RC-2-2-A.
9349	10-24-81	Clean Reactor Vessel Flange.	Cleaned Reactor Vessel Flange per procedure MP-RC-1-3.
9390	10-5-81	Install alignment pins.	Alignment pins installed per procedure MP-RC-6-5-B.
9312	10-7-81	Remove spent fuel pool canal gate.	Removed gate and stored on rack in transfer canal per procedure FH-16-1.
12086	10-8-81	Remove Reactor Vessel Head Lift Rig.	Head Lift Rig removed and stored.
9447	10-14-81	Remove missile shields.	Missile shields removed and placed in storage area.

Approved By:

  
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Manager - Fort Calhoun Station