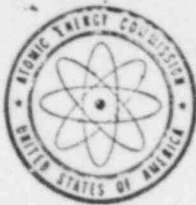


J. P. O'REILLY



UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
970 BROAD STREET
NEWARK, NEW JERSEY 07102

201 645-3942

DEC 15 1970

Consolidated Edison Company of New York
4 Irving Place
New York, New York

Attention: Mr. William J. Cahill, Jr., Vice President

Gentlemen:

This letter relates to the discussion Messrs. Whitesell and Varela of this office held with Mr. E. J. Dadson of your staff during the inspection of November 16 and 17, 1970, regarding the construction activities authorized by AEC Construction Permit No. CPPR-62.

As noted during the discussion, apparent deficiencies were identified involving items not in conformance with Appendix B to Title 10, Code of Federal Regulations, Part 50, entitled, "Quality Assurance Criteria for Nuclear Power Plants", or which may otherwise raise questions concerning adequacy of construction. These items are as follows:

Criterion V, Appendix B, 10 CFR 50, entitled, "Instructions, Procedures, and Drawings", states in part:

"Activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, or a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures, or drawings. . . ."

Contrary to the above, the inspectors found:

1. Procedures or instructions relevant to heating the reactor vessel lifting beam and means of determining the surface temperature of this beam were not prepared, although the procedure for "Handling and Setting of IP-3 Reactor Vessel" stipulates that the lifting beam must not be used when its surface temperature falls below 70° F based on NDTT considerations.
2. The log, showing gas pressures, condition of covers, and visitors to the area, is not being maintained as required by the Wedco procedure, "Reactor Vessel and Reactor Vessel Head Receipt Inspection and Security During Storage."

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3. The Wedco "Procedure for Handling the Reactor Vessel Skid" references, among others, a "Procedure for Installing the Lifting Beam" and a "Procedure for Setting the Reactor Vessel." There was no evidence that either of these procedures were being developed.
4. Wedco's procedure WQA-4.0, part 2.3.3-a, states in part:

"All records and documentation from the supplier must be on site and acceptable before the item is fully accepted."

Contrary to this procedure, the reactor vessel has been released for installation prior to receipt of the N-1, "Data Report Form."

Please provide us, within 30 days, with your comments concerning these items and any steps which have been or will be taken to correct them and to minimize recurrence, including any appropriate changes that have been or will be made to your quality assurance program.

Should you have any questions concerning the matters discussed in this letter, you may communicate directly with this office.

Very truly yours,

Robert M. Howard
for Robert W. Kirkman
Director

CO: L: EME

MEMO ROUTE SLIP

Form AEC-93 (Rev. May 14, 1947) AI 0240

See me about this
Note and return

For conference
For sig

For action
For information

TO (Name and unit) J. P. O'Reilly <i>WJ</i> Chief, RI&EB, CO:HQ	INITIALS DATE	REMARKS CONSOLIDATED EDISON COMPANY INDIAN POINT 3 <i>Reg File</i> 50-286
TO (Name and unit) A. Giambusso, CO L. Kornblith, CO	INITIALS DATE	REMARKS Forwarded herewith is a copy of a CDN, issued to the licensee 12/15/70, resulting from in- formation contained in CO Report No. 286/70-6.
TO (Name and unit) R. Engelken, CO	INITIALS DATE	REMARKS Report is presently in reproduction and will be forwarded for information.
FROM (Name and unit) E. M. Howard <i>emh</i> Sr. Reactor Inspector CO:I	INITIALS DATE	REMARKS <i>Memo</i>
PHONE NO. 3942	DATE 12/17/70	REMARKS

USE OTHER SIDE FOR ADDITIONAL REMARKS