DUKE POWER COMPANY

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A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

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USNRC REGION II
ATLANTA, GEORGIA

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October 16, 1981

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U.S. NUCLEAR REGULATORY COMMISSION

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Re: IE Bulletin 79-14
Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. O'Reilly:

As requested by Mr. W. P. Ang of your office during his audit of September 30 - October 2, 1981, per Item 4C of IE Bulletin 79-14, please find attached an update of Duke Power Company's responses previously submitted to your office on October 29, 1979, November 6, 1979, December 7, 1979, January 24, 1980, and June 11, 1980 for the subject bulletin.

I declare under penalty of perjury that the statements set forth herein are true and correct to the best of my knowledge, executed on October 16, 1981.

Very truly yours,

A. C. Thies

A. C. Thies

JLJ/php
Attachment

ELECTRICITY FOR A PROGRESSIVE PIEDMONT
75th Anniversary
DUKE POWER 1904-1979

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Duke Power Company
Oconee Nuclear Station
Update to IE Bulletin 79-14 Response

On June 11, 1980, Duke Power Company submitted an updated response to Items 2 and 3 of IE Bulletin 79-14. This summary completed our response to Items 2 and 3 of the subject bulletin. In addition to the description of the seismic input nonconformances identified during the site inspections, an evaluation of the effect of each nonconformance upon system operability under specified earthquake loadings was provided as required by Item 4A of the bulletin. There has been no change to that summary.

After a review of the information obtained from the site surveillance and the original analysis and design documents contained in our files, it was decided to reanalyze approximately 100% of the piping stress analysis problems and approximately 100% of the pipe supports. During this reanalysis, modifications are specified as required to assure that the piping and the pipe supports conform to the requirements and design allowables described in the Oconee Final Safety Analysis Report. This reanalysis and modification work is intended to satisfy the requirements of Items 4B, 4C, and 4D of the subject Bulletin. The reanalysis work commenced as soon as the first completed system information package was transmitted to the Design Engineering Department from the Oconee Site Surveillance Organization. The manpower involved in this phase of the work has peaked at over two hundred and fifty (250) individuals and is still continuing. Modification packages are being issued to the Oconee site 79-14 Bulletin Organization for implementation. This implementation work has been slowed by the large amount of work required by other regulatory requirements (e.g., IE Bulletins 80-11 and 81-01, TMI Required Modifications for the Reactor Vessel Head Vent and Containment Hydrogen Sample System, fire protection changes, and other operating control changes), as well as the work associated with the ten year inservice inspections, core barrel repairs, Reactor Coolant Pump inspections, and the Stand-by Shutdown Facility tie-ins. During the current Unit 1 outage, the work associated with these modifications has been in excess of 333,000 man-hours with more than 500 individuals involved in triple shift work schedules.

During the inspection conducted in our offices by Mr. W. P. Ang of the USNRC Region II on September 30 through October 2, 1981, it was requested that a status of the work associated with IE Bulletin 79-14 for Unit 1 be provided. To date, the piping stress reanalysis has been completed for all systems or portion of systems associated with the subject bulletin. Reanalysis of all pipe supports located in the normally inaccessible regions has been completed. Reanalysis of all pipe supports located in the normally accessible regions is still ongoing with a scheduled completion date prior to the end of the current unit outage. The enclosed table summarizes the work associated with the normally inaccessible regions of Unit 1 and indicates the work projected to be completed during the outage.

Also during Mr. Ang's inspection, the subject of the structural overlap technique utilized in our pipe stress analysis was reviewed. We have committed to provide a complete description of the technique used and also to provide a benchmark of the technique against the McGuire Nuclear Station and NUREG 51357 techniques. This information will be provided with our final IE Bulletin 79-14 response.

UNIT 1
Normally Inaccessible Region
Status as of October 15, 1981

Total Number of Required Support Modifications	921
Total Number of Modifications Projected to Be Completed during the Current Outage	635
Total Number of Support Modifications on Supports with Expansion Anchors	206
Total Number of Modifications Projected to Be Completed during the Current Outage on Supports with Expansion Anchors	206