TABLE 3.3-3 (Continued)

TABLE NOTATION

#Trip function may be bypassed in this MODE below P-11.

###The channel(s) associated with the protective functions derived from the out of service Reactor Coolant Loop shall be placed in the tripped mode.

ACTION STATEMENTS

ACTION 13 - With the number of OPERABLE Channels one less than the Total Number of Channels, be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.

ACTION 14 - With the number of OPERABLE Channels one less than the Total Number of Channels:

a. Below P-11 or P-12, place the inoperable channel in the tripped condition within 1 hour; restore the inoperable channel to OPERABLE status within 24 hours after exceeding P-11 or P-12; otherwise be in at least HOT STANDBY within the following 6 hours.

b. Above P-11 and P-12, place the inoperable channel in the tripped condition within 1 hour; operation may continue until performance of the next required CHANNEL FUNCTIONAL TEST.

ACTION 15 - With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT SHUTDOWN within the following 12 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.

ACTION 16 - With the number of OPERABLE Channels one less than the Total Number of Channels:

a. Below P-11 or P-12, place the inoperable channel in the bypass condition; restore the inoperable channel to OPERABLE status within 24 hours after exceeding P-11 or P-12; otherwise be in at least HOT SHUTDOWN within the following 12 hours.

PROPOSED WORDING

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$, 0.10 percent by weight of the containment air per 24 hours at P_a, (38.3 psig), or

b. A combined leakage rate of $\leq 0.60\ L_a$ for all penetrations and valves subject to Type B and C tests as identified in Table 3.61, when pressurized to P_a.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N 56.8-1981:

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at P_a (38.3 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

PROPOSED WORDING

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. If any periodic Type A test fails to meet .75 L_a, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet .75 L_a, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet .75 L_a at which time the above test schedule may be resumed.

c. The accuracy of each Type A test shall be verified by a supplemental test which:

1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within 0.25 L_a.

2. Has a duration sufficient to accurately establish the change in leakage for between the Type A test and the supplemental test.

3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a (38.3 psig).

d. Type B and C tests shall be conducted with gas at P_a (38.3 psig) or as specified by ANSI N56.8-1981 at intervals no greater than 24 months except for tests involving:

1. Air locks

2. Penetrations using continuous leakage monitoring systems, and

3. Valves pressurized with fluid from a seal system.

e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

f. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least 1.10 P_a (42.1 psig) and the seal system capacity is adequate to maintain system pressure for at least 30 days.

TABLE 3.6-1 (Contined)

VALVE NUMBER INSIDE/OUTSIDE		FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME (Sec) INSIDE/OUTSIDE
C. Containment Purge and Exhaust				
1. VS-D-5-3B	VS-D-5-3A	Containment Purge Exhaust	No	8(1) 8(1)
2. VS-D-5-5B	VS-D-5-5A	Containment Purge Sypply	No	11(1) 8(1)
3.	VS-D-5-6	Containment Purge Vacuum Breaker	No	N/A N/A
D. Manual				
1. 1CH-181	MOV-CH308A	Seal Injection Water to RC Pump	No	N/A N/A
2. 1CH-182	MOV-CH308B	Seal Injection Water to RC Pump	No	N/A N/A
3. 1CH-183	MOV-CH308C	Seal Injection Water to RC Pump	No	N/A N/A
*4. MOV-CH142		Reactor Coolant System Letdown	Yes	N/A N/A
*5 MOV-CC112B3	1CCR-252	CCW from RHR Hx & RHR Pump Seal Coolers	Yes	N/A N/A
*6 MOV-CC112A3	1CCR-251	CCW from RHR Hx & RHR Pump Seal Coolers	Yes	N/A N/A
*7 MOV-CC112A2	1CCR-247	CCW to RHR Hx & RHR Pump Seal Coolers	Yes	N/A N/A
*8 MOV-CC112B2	1CCR-248	CCW to RHR Hx & RHP Pump Seal Coolers	Yes	N/A N/A
#9	MOV-FW-151A	Auxiliary Feedwater	Yes	N/A N/A
#10	MOV-FW-151B	Auxiliary Feedwater	Yes	N/A N/A
#11	MOV-FW-151C	Auxiliary Feedwater	Yes	N/A N/A
#12	MOV-FW-151D	Auxiliary Feedwater	Yes	N/A N/A
#13	MOV-FW-151E	Auxiliary Feedwater	Yes	N/A N/A
#14	MOV-FW-151F	Auxiliary Feedwater	Yes	N/A N/A
#15	MOV-RW-104A	Riverwater to Recirc. Spray Hx	Yes	N/A N/A
#16	MOV-RW104C	Riverwater to Recirc. Spray Hx	Yes	N/A N/A
#17	MOV-RW104B	Riverwater to Recirc. Spray Hx	Yes	N/A N/A
#18	MOV-RW104D	Riverwater to Recirc. Spray Hx	Yes	N/A N/A

(1) Applicable in mode 6 only. The provisions of specification 3.0.4 are not applicable. The Containment Purge Exhaust and Supply valves will be locked shut during operation in modes 1, 2, 3, and 4.

REFUELING OPERATIONS

CONTAINMENT BUILDING PENETRATIONS

LIMITING CONDITION FOR OPERATION

3.9.4 The containment building penetrations shall be in the following status:

a. The equipment door closed and held in place by a minimum of four bolts.

b. A minimum of one door in each airlock is closed, and

c. Each penetration providing direct access from the containment atmosphere to the outside atmosphere shall be either:

1. Closed by an isolation valve, blind flange, or manual valve, or

2. Exhausting at less than or equal to 7500 cfm through OPERABLE Containment Purge and Exhaust Isolation Valves with isolation times as specified in Table 3.6-1 to OPERABLE HEPA filters and charcoal absorbers of the Supplemental Leak Collection and Release System (SLCRS).

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or movement of irradiated fuel in the containment. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.4.1 Each of the above required containment penetrations shall be determined to be in its above required condition within 150 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS or movement of irradiated fuel in the containment.

4.9.4.2 The containment purge and exhaust system shall be demonstrated OPERABLE by:

a. Verifying the flow rate through the SLCRS at least once per 24 hours when the system is in operation.

b. Testing the Containment Purge and Exhaust Isolation Valves per the applicable portions of Specification 4.6.3.1.2, and

c. Testing the SLCRS per Specification 4.7.8.1.

REFUELING OPERATIONS

CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.9 The Containment Purge and Exhaust isolation system shall be OPERABLE.

APPLICABILITY: MODES 5 and 6*

ACTION:

With the Containment Purge and Exhaust isolation system inoperable, close each of the purge and exhaust penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provisions of Specification 3.0.3 are not applicable.

*NOTE: The dampers must meet the isolation requirements of Table 3.6-1 for Mode 6 only.

SURVEILLANCE REQUIREMENTS

4.9.9 The Containment Purge and Exhaust isolation system shall be demonstrated OPERABLE within 150 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment Purge and Exhaust isolation occurs on manual initiation and on a high-high radiation signal from each of the containment radiation monitoring instrumentation channels.

REACTOR COOLANT SYSTEM

STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. Steam generator tubes shall be examined in accordance with the method prescribed in Article 8 - "Eddy Current Examination of Tubular Products, " as contained in ASME Boiler and Pressure Vessel Code, Section V - "Non-destructive Examination," and referenced in ASME Boiler and Pressure Vessel Code - Appendix IV to Section XI as amended in the Summer 1976 Addenda "Inservice Inspection of Nuclear Power Plant Components." The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

b. The first inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

1. All nonplugged tubes that previously had detectable wall penetrations (greater than 20%), and

2. Tubes in those areas where experience has indicated potential problems.

PROPOSED WORDING

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

a. At the first refueling outage, dye penetrant inspections shall be performed on all six reactor vessel nozzles and volumetric inspections shall be performed on at least two nozzles in the same manner as that conducted in the baseline inspection.

b. At the second refueling outage, a dye penetrant examination shall be performed on all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle to safe end welds from the outside diameter.

At the third refueling outage, a dye penetrant examination shall be performed or all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle to scfe end welds from the outside diameter. Ultrasonic examination shall be performed on the three outlet nozzle to safe end welds from the inside diameter.

c. Defects found during these inspections shall be evaluated to determine their significance and reported to the Commission pursuant to Technical Specification 6.9.1.8.c.

PROPOSED WORDING