

RELATED CORRESPONDENCE

AAM 11/2/81

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMISSION
ATOMIC SAFETY AND LICENSING BOARD

DOCKETED
USNRC

Before Administrative Judge Gary L. Milhollin ⁸¹ NOV -4 A10:14
as Special Master *er*

OFFICE OF SECRETARY
DOCKETING & SERVICE
BRANCH

In the Matter of)	
)	
METROPOLITAN EDISON COMPANY)	Docket No. 50-289
)	(Restart)
(Three Mile Island Nuclear)	(Reopened Proceeding)
Station, Unit No. 1))	

AAMODT TESTIMONY OF
PROFESSOR CHARLES HOLZINGER
AND PROFESSOR BRUCE MOLHOLT

8111060377 811102
PDR ADOCK 05000289
G PDR

SUMMARY

The purpose of this testimony is to shed light on the extent of cheating in Reactor Operator and Senior Reactor Operator examinations at the TMI-1 facility since the March 1979 Unit 2 accident. The extent of cheating is viewed from two perspectives, the number of operators who may have been involved and the degree of cooperation between O and W.

TESTIMONY OF PROFESSOR BRUCE MOLHOLT

Summary statement

Senior reactor operators O and W cheated extensively on their requalification tests 24 April 1981. There was exact coincidence in 87 percent of their answers, right down to the misspellings. Apparently, sometimes O copied W, sometimes W copied O.

There were 67 separate questions or parts of questions addressed on the SRO "B" exam. I have not graded 5 of these, which are figures. Of the remaining 62 questions or parts of questions, 54 were answered by O and W in identical or nearly identical fashion. I conclude that these two senior reactor operators collaborated on 87 percent of their answers.

A few examples of identical copy are given below:

Answers to question N 2 b

W N. 2 B You cool down the plant by increasing your steaming rate in the OT-56 and lowering the pressure which lowers the saturation temperature.

O N- 2b - You cool down the plant by increasing your steaming rate in the OT-56 and lowering the pressure which lowers the sat temp accordingly.

Answers to question N 4 c

W (C) I would immediately select the two 2 operable NSCCW pumps for E.S. Making sure of physical separation. I would call the duty superintendent, supervisor of ops

O 4c we would immediately select the two 2 operable NSCCW pumps for ES with the 43 selector switch. I would call the duty superintendent, the supervisor of ops

A few additional copies of nearly identical copy are given below in which the major change is that O writes in 2d person, W in 3d person:

Answers to question J 2

W	An RWP for airborne is needed when the concentration is $> 3 \times 10^{-11}$ uc/cc with no alpha present, or when you are $> 25\%$ of MPC.
O	You need an RWP for airborne when the concentration is $> 3 \times 10^{-11}$ uc/cc with no alpha present or when you are $> 25\%$ of MPC.

Sometimes they misspell the same word:

W	in hand. When the rod drops you will get an asymmetrical rod alarm &
Question L3a	in hand. When the rod drops you will get an asymmetrical rod alarm &

Insight is gained as to who is copying whom when O can't read W's handwriting and "collapsing" becomes "colloping":

W	Cause the pressure to increase thus Colloping the steam in the pressure.
Question N5b	Cause the pressure to increase thus colloping the steam in the pressure.

But then, sometimes W can't read O's handwriting (which is generally worse) either, as when "vacuum," with an ambiguous first "u" by O, becomes "vaccum" in W's answer:

Question N3b	You won't be as sub cool as you were with a better vacuum	You won't be as sub cooled as you were with a better vaccum
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Sometimes it appeared to me as if W were giving answers orally to O containing words which O can't spell. These words were then spelled correctly and legibly in W's answer, but misspelled in O's answer. Examples are:

<u>Part</u>	<u>Question</u>	<u>W</u>	<u>O</u>
I	6a, c 6c	xenon affected	zenon effected
K	1a 3	asymptotically affect	acsomptically effect
L	4a	orifice	orfice

These I believe to be examples where W is giving answers to O, perhaps orally. At least once O can't understand what W is driving at at all, undoubtedly here in a written answer and substitutes his own word rather than copy something nonsensical:

<u>Part</u>	<u>Question</u>	<u>W</u>	<u>O</u>
M	2	collorates (for correlates)	determines (!)

And, at least once, W copies O's misspelled word as something else:

<u>Part</u>	<u>Question</u>	<u>W</u>	<u>O</u>
J	5a	injecting (food!)	injesting (for ingesting)

Summary statement

Senior reactor operators O and W cheated extensively on their requalification tests 24 April 1981. There was exact coincidence in 87 percent of their answers, right down to the misspellings. Apparently, sometimes O copied W, sometimes W copied O.

There is little evidence that the other four senior reactor operators sitting in this room, but not at the same table, were involved in this cheating by operators O and W.

BRUCE MOLHOLT, Ph.D.

Birth- November 5, 1940, Menosha, WI

Ph.D.- 1967, Indiana University, Bloomington, IN (Microbiology)

Postdoctoral Fellowships

1967-69 USPHS, Department of Microbial Genetics, Karolinska Institutet, Stockholm, Sweden.

1972-73 NATO, Department of Molecular Genetics, UNiversity of Ghent, Ghent, Belgium.

Positions

1969-72 Asst Professor of Microbiology, University of Kansas, Lawrence, Kansas.

1973-74 Charge de Recherches, Department of Molecular Biology, University of Geneva, Geneva, Switzerland.

1974-75 Research Associate, Department of Molecular Genetics, University of Heidelberg, Heidelberg, West Germany.

1975-76 Exchange Scientist of US National Academy of Sciences, Institute of Biochemistry, Prague, Czechoslovakia.

1976-78 Associate Professor of Microbiology and Surgery, Medical College of Wisconsin, Milwaukee, Wisconsin.

1979-
present Science Director, Environmental Cancer Prevention Center, Philadelphia, Pennsylvania.

Publications

Twenty articles on molecular genetics.

Genetic Predisposition to Cancer in Man (with A.G. Knudson).

STATEMENT OF PROFESSOR CHARLES HOLZINGER

The Aamodts received in response to discovery requests of the Licensee the PQS and ATTS RO and SRO mock exams, the Category T and Category T makeup exams, and weekly cold licensing requalification exams. The PQS and ATTS tests were provided lacking information with regard to seating arrangements, and except where the exams were dated, no indication of what candidates sat together in a given room. A further hindrance to an investigation of collusion among examinees was the fact that in some undetermined number of instances the Category T and weekly exams were take-home, unproctored and/or open book. Nevertheless, several groups of exams were assembled for analysis which, after a question-by-question analysis, yielded sufficient numbers of examples of similar responses to support the view that collusion of some sort had occurred. There were no fewer than 14 candidates who appeared to be engaged in collusion of some sort.

These forms of collusion did not follow the blatant mode of copying observed in O and W. However, since the essay-type answers to questions on the papers were characteristically very dissimilar between candidates, similarities between essay type responses, especially when there were multiple examples of such similarities between two individuals, indicate a high probability that collusion of some sort has taken place.

Several examples follow.

RELATED CORRESPONDENCE

ATTS RO EXAM GIVEN 4/1/81--FOLLOWING SEVEN PAGES.

Analysis of this set of examinations shows collusion among U, B and Q. See the attached reproductions of Alc (page 1), A3 and A4 (page 2), B2 (pages 3 and 4) and C1 (pages 5 and 6). It should be noted that overall, any given set of essay-type answers are characteristically dissimilar, emphasizing the uniqueness of any similar pairs of answers. Note the distinctly different answers of three randomly selected papers (HH, BB, E) to questions Alc (page 1) and B2 (page 4).



Charles H. Koenigsberg

RELATED CORRESPONDENCE

Fields of Specialization:

Area: North America
Subjects: Human Biology and Evolution; The American Indian; Psychological Anthropology

Courses currently taught: Evolution of Man; Psychological Anthropology; The American Indian; Anthropological Theory

Education:

Franklin and Marshall College, 1939-1941 (No degree)
University of Chicago, 1945-1949
Degree: M.A. in Sociology
Thesis: "Acceptance-Rejection of Belief in Equality of Opportunity"
Harvard University, 1951-1961; 1958; Department of Social Relations

Teaching Positions:

Harvard University, Department of Social Relations, Teaching Fellow, 1953.
Franklin and Marshall College, Department of Sociology and Anthropology
Instructor, 1949-1953
Assistant Professor, 1953-1958
Associate Professor, 1958-1966
Franklin and Marshall College, Department of Anthropology
Associate Professor and Chairman of the Department, 1966-1968
Professor and Chairman of the Department, 1968 to present
Stanford University, Visiting Associate Professor of Anthropology, summer
University of North Carolina, Visiting Associate Professor of Anthropology,
summer 1966.

Publications:

"The Ebough Site: A Susquehanna Valley Cemetery," in Eastern States Archeological Federation Bulletin, No. 17, January 1958.
With John Witthoft and W. Fred Kintsey, "A Susquehanna Valley Cemetery: The Ebough Site," in Susquehanna Valley Miscellany, published by the Pennsylvania Historical and Museum Commission, Commonwealth of Pennsylvania, 1959.
"Some Observations on the Persistence of Aboriginal Cherokee Personality Traits," in Symposium on Cherokee and Iroquois Culture, Smithsonian Institution, Bureau of American Ethnology Bulletin 280, Washington, 1961.
"Archeological Report, Linnelick Cemetery Indian Site, Montgomery and Chester Counties, Pa.," Archaeology No. 81697, York, Pa., 1971.
"Archeological Report, Union Cemetery Indian Site, Lancaster County, Pa.," Archaeology No. 82780, York, Pa., 1971.

Editorial:

Journal of Anthropological Society of America
Journal of the American Society for the History of Anthropology
Journal of the American Society for Human Genetics
Journal of the American Society for Human Genetics
Journal of the American Society for Human Genetics
Journal of the American Society for Human Genetics
Journal of the American Society for Human Genetics
Journal of the American Society for Human Genetics



Q) negative over core life due to boron depletion
 & buildup of more resonance capture peaks (RC) due
 to ^{240}Pu buildup, thus increasing the stability of
 the reactor

c) Mod coefficient becomes more
 core life due to less boron poison product buildup.

a) Over core life Pu-240 builds into the core
 while U-238 remains constant. With more Pu-240
 this will increase the chances of β capture in
 resonance peaks while the β is being moderated.
 Over core life mod temp will get more neg due to
 increased Pu-240 buildup and decreased boron content
 lower boron gives less + feedback when coolant heats up.

c. At operating temperatures and BOL Boron concentration
 we have a $-0.77 \times 10^{-4} \text{ } \Delta k/k / \text{ } ^\circ\text{F}$ reactivity coefficient value
 and $-2.63 \times 10^{-4} \text{ } \Delta k/k / \text{ } ^\circ\text{F}$ with EOL Boron Concentration (217 ppm)
 the negative reactivity change per $^\circ\text{F}$ increase in moderator temperature
 becomes more negative due to the increase in Pu-240 in the
core, the boron reduction, and the increase in additional fission
product inventory with resonance capture peaks.

A c) This effect is called moderator temp coeff
 and gets more negative over core life.
 BOL $-0.77 \times 10^{-4} \text{ } \Delta k/k / \text{ } ^\circ\text{F}$ EOL $-2.63 \times 10^{-4} \text{ } \Delta k/k / \text{ } ^\circ\text{F}$ Due to
 boron dilution and buildup of Pu-240 + other
 fission products fission to the RC peaks.

$\frac{1.0}{1.0}$ \square are shut down or would be shut down if all rods were to drop into the core except the highest worth rod was stuck at the top and Group rods were to stay right where they are.

A-3) a). The amount of reactivity which the reactor is shut down by or would be shut down by from its present conditions assuming all rods inserted except group X and the

$\frac{1.0}{1.0}$ \square shutdown margin is the amount of reactivity which must be added to the core to be critical with no insertion of βp , and the rods ~~are not~~ stuck out. What you called $\frac{.5}{1.0}$ causes did to be erroneous.

A-4) Z_0 in building due to the decay of I and λ and is being removed by decay or absorption of n

$\frac{.25}{2.0}$ \square The rate of build up of I or λ decay is equal to the λ of I (plus λ plus decay of X or n)

B
27
2/2

Gen into a downcomer region where it is preheated on the way down to the bottom of the OTSG. It flows up the OTSG where it is heated undergoes sit & superheated & out the top.
Emergency F.W. enters near the top of the OTSG right onto the tube bundle to cool the RCS.

1) a) the main feed is introduced into the OTSG by the main feed ring which is ~50% up the OTSG on the outside with nozzles that penetrate the OTSG and spray downward into the downcomer allowing some preheat of the water by evaporating steam.

2) the emergency feed is introduced by the emergency feed ring at the top of the area with feed nozzles spray water directly onto the tubes of the OTSG.

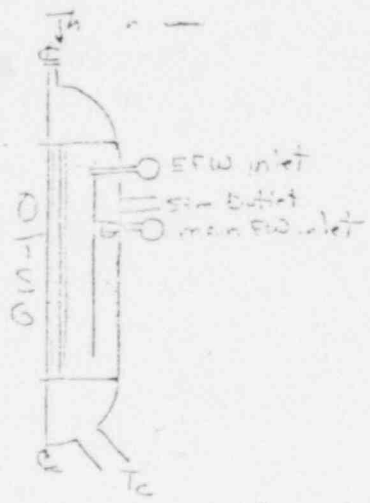
3-2 of main feed enters a feed ring about the center of the OTSG just below where the main steam lines top off. It enters at the top of the downcomer region of OTSG which located on outer circumference of OTSG (but inside of OTSG) & draws off a part of steam from the generator to preheat the F.W. prior to reaching the tube sheet & tubes of OTSG thus limiting the thermal stress on the tube sheet & tubes & inc efficiency & is used during normal & S/W @ 45

watch your welding!

... steam comes out from the upper part of the tubes (which preheats feed to sat. steam) the time it reaches the tube sheet) and is directed down along the outside (of the tubes) and up through the tube sheet, and up around the tubes to be heated by primary water. Every feed is introduced much higher into the steam generator, and it discharges directly on to the OTSG tubes then steam. It is not preheating involved.

B-2

A)
4/1/21
ES
32270



EFW Enters Tube Area High in OTSG
Main F.W. Enters Down Corner

B-2 a) Auxiliary feedwater enters the OTSG higher than main feedwater. Aux feed enters the upper end of the OTSG and is sprayed directly on the tubes. Main feedwater enters lower and goes down the downcomer before entering the lower tube section. This preheats the water before entering lower tube area. Auxiliary steam used for preheat x
b) Aux feed is arranged in this way to promote

5

NOTE - U DOES NOT DERIVE
 $\Delta P = \left[\frac{k_2-1}{k_2} - \frac{k_1-1}{k_1} \right]$ BUT PUTS .006 - .012
 OUT OF (1000000?)

95% on GP 5

$\Delta P = (k_2 - k_1)$ → WRONG
 $\Delta P = .006 - .012$
 $\Delta P = .006 \times 100 = 0.6\%$

2
2

u

$P_1 = .012$ $P_2 = .006$

$P = \frac{k}{k-1}$ $P = \frac{k}{k-1}$

$P_1 =$
 $P_2 =$
 $\Delta P =$

$P = \frac{k}{k-1}$
 $\frac{k_1}{k_1-1} = \frac{k_2}{k_2-1}$

(c1) 50% GP 5 $k_1 = .983$ 1500 cps
 95% GP 5 $k_2 = .994$ 3000 cps

Q

C-1 $\frac{1.6}{11} = 75\%$

-1 $\frac{CR_1}{CR_2} = 2 \cdot \frac{(1-K_2)}{(1-K_1)}$ ✓

$CR_1 = 1500$ cfs $K_1 = .988$
 $CR_2 = 3000$ cfs

$\frac{1.6}{2}$

$\frac{1500}{3000} = \frac{(1-K_2)}{(1-.988)}$

$\frac{1}{2} = \frac{1-K_2}{.012}$

$.006 = 1-K_2$

$K_2 = 1 - .006 = .994$

$P_2 = \frac{K_2 - 1}{K_2} = \frac{.994 - 1}{.994} = -.006036$

$P_1 = \frac{K_1 - 1}{K_1} = \frac{.988 - 1}{.988} = -.012145$

$\Delta P = P_2 - P_1 = (-.006036) - (-.012145)$

$\Delta P = .00611$

↑ ↑
L should be 70
.611%

→ G_5 @ 50% = -2.7

$(-2.7) + (.00611) = -2.69389$ $(-2.7) + (.6) = -2.1$

∴ G_5 new position is 51% $-2.1\% \approx 95\%$ on G_5

F7) Because you will carbon up the
valve and also unused fuel will
accumulate in the exhaust manifold
and eventually ignite causing a fire.
→ diesel engine does not have valves!



IF IT DOESN'T, ITS A PRETTY
NOVEL ENGINE!

RELATED CORRESPONDENCE

FOLLOWING 6 PAGES.

EXAMPLES OF COLLUSION ATTON/6

A, P, V & T DIV AITS RO EXAM 4/2/81



9.4/12

78.3

B1a main FW block (FW-V-5) opens when SV (FW-V-16) is 90% open and closes when FW-V-16 is ~~90%~~ 70% closed when in auto

1.5/2 main FW control valve (FW-V-17) opens after SV FW control valve is open when in auto

P-4/2/21

9.1/12 = 75.8

B-1 a) The main FW Block valve opens at 90% open on the startup FW control valve automatically and close automatically at ~~90%~~ 70% on the startup FW control valve

1.75/2

1.8/1

SAME ELEMENTS, ORDER
" UNIQUE ERROR

0.5/1

P

of logging of the results
 in the section from the
 R.B. sump by Foreign
~~and~~ material could restrict
 flow, or shifting section
 to the sump with more
 than 3' in the BWST
 could result in insufficient
 Net positive suction head
 for the decay heat pumps
 High activity in sump fluid

(P-2)

V

- Failure of DH-PIA-2A
- Failure of DH-V-6-A or B to open DH section from RB sump
- Failure of DH-V-7-A or B to open DH to MV pump section
- Failure of DH-V-5-A or B to close DH section from sump
 contaminated sump fluid

0.5/1

ELABORATE, SIMILAR ELEMENTS,

1/2 RIGHT.

processes for the respective sides versus S.P. + bias. Normally the bypass valves with controlling at CRS PSIG + 10 bias ^{OK} at low loads will start up. As the turbine starts to take steam, the pressure drops and the bypass valves close to maintain pressure. When the bypass valves get shut and ULD is $\geq 15\%$, the bias will auto shift to + 75 psig. 1 other condition

A

225/3

temp increase

B# The turbine bypass valve will close to maintain turb. header press. setpoint plus 10th bias (normally 895 PSIG). The parameter it controls is turbine header press. even though it is seeing str. gen. press. The bias is shifted to +75th when 1. the turb. is reset, 2. ULD is $\geq 15\%$, & 3. All bypass valves are closed

I

not quite true
long response

224/3

D-3

This can be caused by changes in the reference junction temperature where the thermocouple is attached to the circuit since they are calibrated at a certain reference junction temperature if you change it you introduce error.

.5/2

P

This can happen during a LOCA or steam break in the RB as temperatures are elevated under those conditions and the junctions are in the RB.

Physical Damage, Multiple, False Junctions

03

V

.5/2

This type of failure is usually associated with a loss of reference junction against which the thermocouple measures thermionic current due to ambient conditions only. This loss of reference junction would probably be associated with extremely high temp / moisture content conditions such as may exist in RB following a LOCA multiple or false junctions

ATTS - RO EXAM 4/2/81

②

A

5/2

(D.3) we have found two problems at sea about
 like described it is due to high resistance
 of junctions and the other one due to
 reference junction changes changes in the
 environment when the reference junction is placed
 can take place due to loss of contact
 or steam leaks near the junction.
 Multiple or False Junctions - thermal or Mech. Damage.

"A" APPEARS TO HAVE CLEAR UNDERSTANDING
 OF THIS LIMITED VIEW OF THE SITUATION -
 "P" & "V" APPEAR TO HAVE RECEIVED
 THIS INFORMATION FROM "A"

(5)

F-4 a. Prior to starting the 3rd reactor coolant pump must be > Rod oper. curve of figure #1, Have ICCW running, and plot 1/M. Second vent must be complete.

D
25
2.0

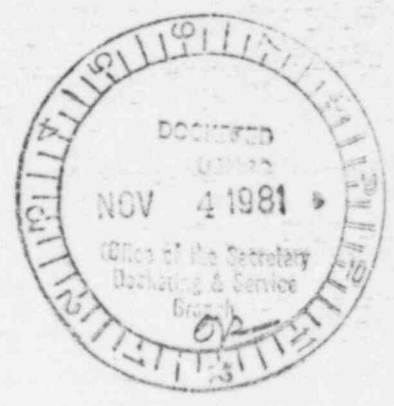
b. After resetting the ^{S.C.A.} ~~Pressure~~ Pressure by stabiles in RPS when coming out of SIP Bypass. Same restrictions apply. See 1102-1 PT 4.0

F.4 Prior to starting the 3rd RPS which actually starts the ^{start} ~~hittop~~ withdrawal you must monitor counts and plot $\frac{1}{M}$ for the withdrawal. SD Bypass Hi Pwr @ 4.25% when RPS pressure = 1650 you must insert rods - increase pressure ^{to 1900 psi} then SECP. Reset the RPS (go out of SIP ^(is pwr) and then stabiles must be withdrawn ^{Hi Pwr to 104.75%} again - watch counts and plot $\frac{1}{M}$. Maintain < 1% SDM

1.0
2.0

(6)

FOLLOWING TWO PAGES
EXAMPLES OF COLLUSION BETWEEN
F AND X ON SRO "A" EXAM.



SRO A
F

Re trip would be reduced due to reduction in temp at EOL would insert a larger amt of positive reactivity.

c) = The Power deficit gets more with core age mainly due to the leakage coefficient becoming more

(4B) ii This makes our shutdown margin smaller after a trip since the ^{core} reactivity after the trip will add ^{to} positive reactivity at EOL ^{than} at

I (4)

c) 1. Power Deficit becomes more negative over core life because it is a function of moderator temp coefficient + the leakage coefficient. Both get more negative over core life due to the build up of Pu-240 + fission product. Another factor causing power deficit to be more negative at EOL than BOL is the flux redistribution that takes place over core life.

SRO A
X

100% secondary (1/2)
 The basis for the 100% is in not receiving system integrity and the limit for 100% is as you don't exceed 1.5 R in 2hr at site boundary on a lead rejection and the container left.

SRO-A
F

J. (c) a) Primary to secondary leak rate - must not be > 10ppm

Secondary system activity must not exceed 1.0 uCi/ml

(1/2)

System integrity 100% limit on substrate will minimize unwanted releases of activity from the secondary plant

The 1.0 uCi/ml limit is based on not exceeding 1.5 R dose at the site boundary on a lead rejection.

SRO-A
X

(2)

RELATED CORRESPONDENCE

FOLLOWING 4 PAGES

EXAMPLES OF COLLUSION BETWEEN

CC & P ON PQS RO EXAM 4/2/80



0.5 liquid is away from the saturation or two phase condition.

A-1-0

sub cooling is that amount in
OF or ΔC that a liquid is below
its saturation temp for the pressure
it is at

F-0.5

B.4.a. Possible passive source of heated water
to recover the core in the event of a loss
of coolant accident until the emergency
injection systems are operating and providing
cooling (heated water) to the core.

CC

B-4-a on a large loop to provide a
possible injection source to recover
the core in the time required to
start HP injection & LP injection

F

- C.5. Solid plant operation indications
- 1) pressure level - full scale
 - 2) wide range pressure - oscillating near safety
relief setpoint
 - 3) pressure temperature less than saturation
due to colder water in pressure (outlet and feed)
 - 4) RCOT level increasing from flowdown
through relief valves

V-0

C-5. To verify relief valves
assuming plant instrumentation
is working OK.

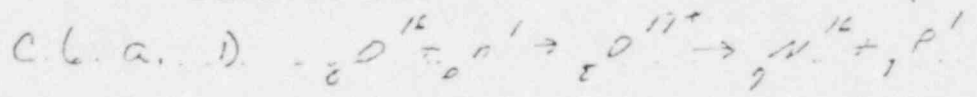
P

P20 level ↑ to 400" followed by
(when level filled) a rapid increase of 100-200 PSI over
saturation pressure for P20 temp.
assuming no leaks in P20 & P21

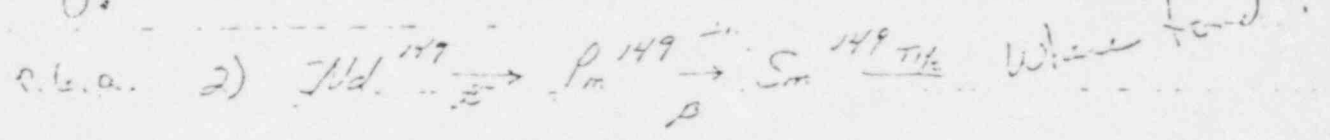
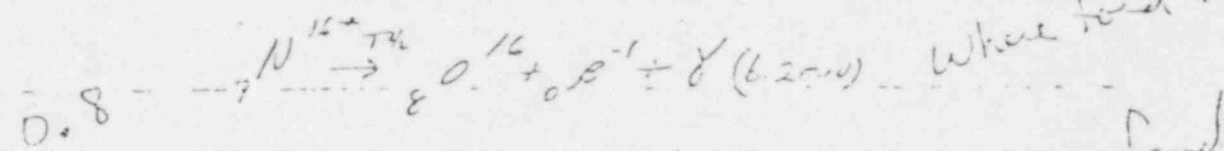
V-0

BEST COPY AVAILABLE

rapidly with a change in μ or λ/D flow.



CC



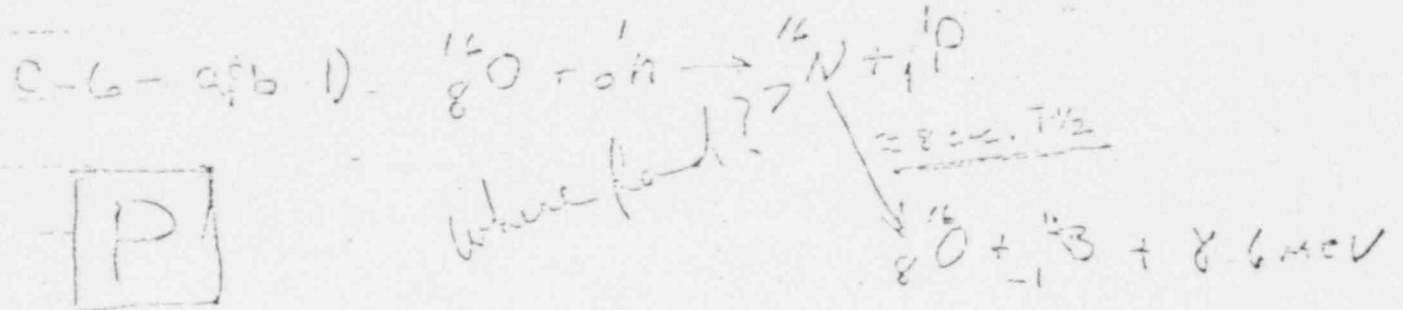
c.6.a. 3) Krypton 85 comes from the fission process as a daughter product

c.6.b. 1. $T_{1/2}$ Trends for $T_{1/2}$ stability of nuclei

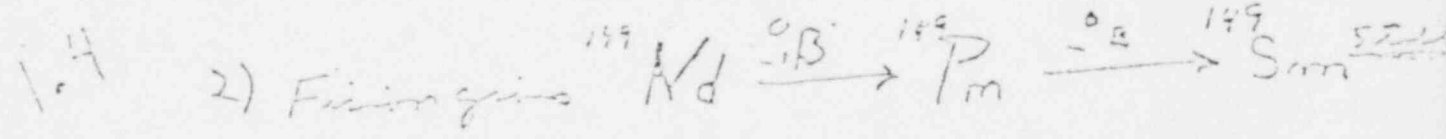
c.6.b. 2. $T_{1/2} \approx 10^{16}$ years for $T_{1/2}$ stability formula

c.6.b. 3. Express.

0.5



P



3) Krypton 85 comes from fission
 $T_{1/2} \approx 10$ years

3)

CC

steam generators on the vertical section
of console 'CC' about 2 feet to the left of
the power range meters. 0.5

D-3-b. 50% on the operating range recorder for the
steam generators on the vertical section
of console 'CC' below the startup range
meters 0.5

D-3-a. " on startup range read on
console C.L. 0.5

D-3-b 50% on operating range read on
0.5 stop recorder console C.L.

NOTE: CCR CC LOOKS
LIKE 'CL' - WHICH P ONLY,
IN THIS GROUP OF TESTS, WROTE

CC

and slow down region. With voiding
there was less moderation and reflection
of neutrons and an increase in leakage
neutrons to be seen by the detectors

(DS) b. Voiding in the core. (Steam is
not as good a reflector & moderator
of σ_a and more escaped to be
seen on the IR detectors)

1.0

FOLLOWING 2 PAGES

LIST EXAMPLES OF COPYING BY O & W
ON ATTS RO & SPO EXAMS

WE HAVE NOT COPIED ACTUAL ANSWERS

BECAUSE OF THEIR EXTREMELY OBVIOUS

SIMILARITIES.



Similarity in O and W on ATTS Exam on 4/3/81

See I 1a } Strong similarity in phrasing
I 1b } makes collusion a necessary conclusion
I 2 }

J 2

J 3

J 4

See that grades seemed to change with

J 5

K 1a

K 6d

L 3

Note some grammatical error, "natural"

L 4

M 2

M 3a

M 3b

N 1

See similarity in spelling error

N 2

N 3

Similarity in response of O and V on ATTS Expt on 4/2/51

A 2 "you are seeing"

A 3

A 6

B 2 a

C 1 c

C 2

C 3

C 4

C 5

D 1 a and b

D 3

D 4

E 3

F 1

F 6

G 1

G 3

G 5

H 1

H 2

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UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

'81 NOV -4 A10:14

ATOMIC SAFETY AND LICENSING BOARD

ers

Before Administrative Judge Gary L. Milhollin
as Special Master

OFFICE OF SECRETARY
DOCKETING & SERVICE
BRANCH

in the Matter of)
)
METROPOLITAN EDISON COMPANY)
)
(Three Mile Island Nuclear)
Station, Unit 1))

Docket No. 50-28^c
(Restart)
(Reopened Proceeding)

SERVICE LIST

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