



Carolina Power & Light Company

October 29, 1981

File: NG-3514(B)

Serial No.: NO-81-1775

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D.C. 20555



BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71

REACTOR CONTAINMENT BUILDING INTEGRATED LEAK RATE TEST

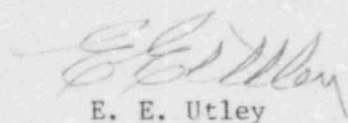
Dear Mr. Denton:

Pursuant to 10CFR50, Appendix J, paragraph V.B.1, twelve (12) copies of the enclosed report entitled "Reactor Containment Building Integrated Leak Rate Test" are transmitted for your use. The report presents the results of the Brunswick Steam Electric Plant, Unit 1 Reactor Building Integrated Leak Rate Test performed in June, 1981. Completion of this test and submittal of this report is also in accordance with Brunswick Unit 1 Technical Specification 4.6.1.2.

The results presented indicated a leakage rate at the upper bound of the 95 percent confidence interval below the allowable leakage rate of 0.375 weight percent per day at the 49 psig test pressure. The results included in the report indicate the containment leakage rate is within acceptable limits and that the containment can perform its design function in the unlikely event of a major accident.

If additional information is needed, please contact us.

Yours very truly,


E. E. Utley
Executive Vice President
Power Supply and
Engineering & Construction

WRM/lr (3842)

Enclosure (12 copies)

cc: Mr. R. A. Hartfield
Mr. T. A. Ippolito

A017
S
11/12
limited
DST

8111040306 811029
PDR ADDCK 05000325
PDR

11 Fayetteville Street • P. O. Box 1551 • Raleigh, N. C. 27602