

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

October 30, 1981

TELEPHONE: AREA 704
573-4083

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: McGuire Nuclear Station
Docket Nos. 50-369, 50-3

Dear Mr. Denton:

The purpose of this letter is to inform you of the action that Duke Power intends to take following notification from Westinghouse of steam generator experience on a Westinghouse non-domestic plant. The steam generators are of similar design to that in the McGuire Unit 1 plant.

A 3-loop unit was shutdown on October 21, 1981 due to primary to secondary leakage. The unit had been operating at power levels greater than 50% for a period of approximately five months.

Inspection of the Number 1 steam generator revealed a tube was leaking on the cold leg side of the steam generator within the preheater section. Eddy current inspection is in progress. The data to date reveals numerous tubes with indications localized within the preheater section at baffle plate locations. The tubes affected are peripheral in rows adjacent to the feedwater inlet. The number of tubes with indications is of the order of 100 per steam generator. The number of tubes with indications reported greater than 50% of wall reduction are:

Steam Generator No. 1 - 21

Steam Generator No. 2 - 15

Steam Generator No. 3 - 10

Westinghouse believes these indications are due to excitation of the steam generator tubes from high fluid velocities and/or non-uniform velocity distribution and that reduction of flow velocity by reducing total feed flow should reduce the potential for vibration. Preliminary evaluations have been performed to determine the effects of continuing to operate the McGuire Steam Generators for the near term.

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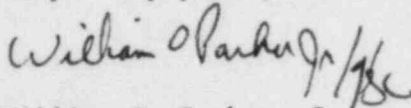
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A conservative evaluation of existing data indicates that operation for one month at 50% power is not expected to cause unacceptable tube wall degradation. Consequently, no potential safety problem is deemed to exist. McGuire is currently in the power ascension test program at 30% power and is scheduled to increase power to 50% within the next few days. It is our intention to complete the testing at the 50% plateau. This is expected to take approximately two weeks. At that time we will evaluate the information available to determine if further testing at higher power levels can be undertaken.

We will keep you advised as additional information becomes available.

Very truly yours,


William O. Parker, Jr.

GAC/smh

cc: M. J. Graham
Resident Inspector
McGuire Nuclear Station

Mr. James Reilly, Director
U.S. Nuclear Regulatory Commission
Region II
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