NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77) LICENSEE EVENT REPORT CONTROL BLOCK (1)(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) -00000-00341 I P B H 1 (2) 0 0 1 0 1111 LICENSE NUMBER CON'T REPORT 0 1 SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During a refueling outage, the performance of TS-30, High and Low Head 0 2 Safety Injection Check Valve Leakage Test, on 10-10-81 indicated leakage 0 3 in excess of the acceptance criteria for 1-853C. This valve serves as 0 4 the first off check valve from the reactor coolant system for the low 0 5 head SI core deluge line. Valves 1-853A and 1-845C could not be tested 0 6 due to the leakage from 1-853C. LER 81-010/01T-0 reports a similar 0 7 This occurrence is reportable in accordance with T.S. 15.6.9.2.A.9. event. 80 0 SYSTEM CAUSE CAUSE COMP VALVE SUBCODE COMPONENT CODE CODE SUBCODE S| F|(11 B (12) A (13) VE C (15) VI A L X (14) A (16) 18 OCCURRENCE REVISION SEQUENTIAL. REPORT EVENT YEAR REPORT NO. CODE TYPE LER/RO NO. REPORT 011 0 T 15 0 NUMBER ACTION FUTURE EFFECT ON PLANT HOURS (22) ATTACHMENT COMPONENT SHUTDOWN METHOD NPRD-4 PRIME COMP. FORM SUB. MANUFACTURER SLIPPLIER Y 23 Z (21) N (25) 5 (26) Z (20 010101 VIOI 81 Z (19 0 Х 18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The leakage noted in this event was about 6 gpm vice the high leak rate 0 noted in LER 81-010/01T-0. The affected line was flushed and 1-853C 1 successfully passed a retest on 10-16-81. With the successful leak rate test on 1-853C, valves 1-853A and 1-845C were tested and met their acceptance criterion. 4 Rn METHOD OF FACILITY OTHER STATUS (30) % POWER DISCOVERY DESCRIPTION (32) H (28) 0 0 0 (29) C (31) Surveillance testing N/A 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED OF RELEASE Z (33) Z (34) N/A N/A 80 4.4 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 0 0 0 37 Z 38 N/A 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 0 (40) N/A R 80 12 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) N/A 8110290484 811022 80 PUBLICITY NRC USE ONLY PDR ADOCK 05000266 DESCRIPTION (45 N 44 N/A PDR 11111 0 69 68 80 414-277-2811 C. W. Fay PHONE: NAME OF PREPARER \_\_\_\_

## ATTACHMENT TO LICENSFE EVENT REPORT NO. 81-015/01T-0

Wisconsin Electric Power Company Point Beach Nuclear Plant, Unit 1 Docket No. 50-266

During the performance of TS-30, High and Low Head Safety Injection Check Valve Leakage Test, on October 10, 1981, leakage in excess of the acceptance criteria for 1-853C was noted. The acceptance criteria for the performance of this leak test was leakage not to exceed 2.5 gallons per minute. The observed leakage was slightly in excess of 6 gallons per minute. Check valve 1-853C serves as the first-off check valve from the reactor coolant system for the low head safety injection core deluge line. Due to the leakage from 1-853C, valves 1-853A (second-off check valve from the reactor coolant system to low head safety injection line) and 1-845C (secondoff check valve from the reactor coolant system to core deluge line) were not tested.

The leakage noted in this event was approximately 6 gallons per minute versus the high leak rate noted in the similar event reported in Licensee Event Report No. 81-010/01T-0. All the other valves in this test passed their acceptance criteria. Backup isolation for 1-853C and the untested valves was provided by motor-operated valves in series. This backup isolation in concert with the plant being in a refueling outage provided assurance that there existed no danger to the public health and safety. The affected line was flushed and 1-853C successfully passed its leak rate test on October 16, 1981. With the successful test of 1-853C completed, valves 1-853A and 1-245C were tested and found to meet their acceptance criterion with essentially no leakage.

This event is reportable in accordance with Technical Specification 15.6.9.2.A.9, "Performance of structures, systems, or components that require remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analysis report on Technical Specification basis". Notification of this event was provided to the NRC Resident Inspector on October 12, and 24-hour written notification was made on October 13.