NRC FORM 366 (7-77)

UPDATE REPORT - PREVIOUS REPORT DATE 2/22/80

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE (2)	A L B R F 1
LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
LICENSE TYPE (4)	4 1 1 1 1
CAT (5)	
CON'T 01 REPORT SOURCE (6)	L
DOCKET NUMBER (7)	0 5 0 0 0 2 5 9
EVENT DATE (8)	0 2 1 0 8 0
REPORT DATE (9)	0 9 2 5 8 1

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With unit 1 in a scheduled refueling outage, unit 2 at steady state power of 1087 MWe,
03 and unit 3 at 1100 MWe, steady state power, notification was received from TVA's
04 Division of Engineering Design that, should a recirculation system line break be
05 followed by a reactor building closed cooling water line break, valve failure, and
06 system drainage, degradation of primary system integrity could result. There was no
07 danger to the health or safety of the public. No previous occurrence. Redundancy
08 does not apply.

09	
SYSTEM CODE (11)	S A
CAUSE CODE (12)	B
CAUSE SUBCODE (13)	A
COMPONENT CODE (14)	Z Z Z Z Z Z
COMP. SUBCODE (15)	Z
VALVE SUBCODE (16)	Z

(17) LER/RO REPORT NUMBER	
EVENT YEAR	8 0
	—
SEQUENTIAL REPORT NO.	0 1 6
	/
OCCURRENCE CODE	0 1
REPORT TYPE	X
	—
REVISION NO.	1

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
G	Z	Z	Z	0 0 0 0	Y	N	Z	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Subsequent evaluation by EN DES determined no immediate safety significancy exists.
11 Evaluation determined two areas where postulated circumferential break locations in
12 the recirculation have potential for RBCCW line impingement. It is EN DES's judgment
13 that RBCCW integrity would not be lost due to impingement and no immediate corrective
14 actions are required.

15	
FACILITY STATUS (28)	H
% POWER (29)	0 0 0
OTHER STATUS (30)	NA
METHOD OF DISCOVERY (31)	A
DISCOVERY DESCRIPTION (32)	Notified by design

16	
ACTIVITY RELEASED (33)	Z
CONTENT OF RELEASE (34)	Z
AMOUNT OF ACTIVITY (35)	NA
LOCATION OF RELEASE (36)	NA

PERSONNEL EXPOSURES

17	
NUMBER (37)	0 0 0
TYPE (38)	Z
DESCRIPTION (39)	NA

PERSONNEL INJURIES

18	
NUMBER (40)	0 0 0
DESCRIPTION (41)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

19	
TYPE (42)	Z
DESCRIPTION	NA

PUBLICITY

20	
ISSUED (44)	N
DESCRIPTION (45)	NA
NRC USE ONLY	

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PDR ADOCK 05000259
S PDR

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
6/04/81

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 80016 R1 Technical Specification Involved 3.7.A.2

Reported Under Technical Specification 6.7.2.A.9 *Date due NRC: NA

Date of Occurrence 2/10/80 Time of Occurrence 1900 Unit 1, 2, 3

Identification and Description of Occurrence:
TVA Division of Engineering Design notified BFNP that design problems existed whereby, should a recirc line break be followed by a RBCCW line break, the RBCCW system could drain and thereby degrade primary containment integrity.

Conditions Prior to Occurrence:
Unit 1 - scheduled refueling outage.

Unit 2 - 1087 MWe, steady state.

Unit 3 - 1100 MWe, steady state.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

None

Apparent Cause of Occurrence:

Design deficiency

Analysis of Occurrence:

There was no danger to the health or safety of the public, no release of activity, no damage to the plant or equipment, and no resulting significant occurrence.

Corrective Action:
None required at this time based on information provided by TVA Engineering Design. This deficiency is still under evaluation and any further actions taken will be provided in a supplemental report.

Failure Data:
NA

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: Will