

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | 0 | H | D | B | S | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 4 | 6 | 7 | 0 | 8 | 2 | 0 | 8 | 1 | 8 | 0 | 9 | 1 | 6 | 8 | 1 | 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | (NP-33-81-60) On 8/20/81 it was discovered that containment isolation check valves
03 | CV 124, CV 125, NN 58, SA 502, and IA 501 had not been tested prior to 8/12/81 per
04 | Technical Specification 4.0.5. There was no danger to the health and safety of the
05 | public or station personnel. Containment integrity was maintained by existing redun-
06 | dant isolation valves in series with these valves.
07 |
08 |

09 | SYSTEM CODE | S | D | 11 | CAUSE CODE | D | 12 | CAUSE SUBCODE | Z | 13 | COMPONENT CODE | V | A | L | V | E | X | 14 | COMP. SURCODE | C | 15 | VALVE SUBCODE | Z | 16 |
7 8 9 10 11 12 13 14 15 16
17 | LER RO REPORT NUMBER | 8 | 1 | 21 | 22 | SEQUENTIAL REPORT NO. | 0 | 5 | 0 | 27 | OCCURRENCE CODE | 0 | 3 | 28 | 29 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32 |
ACTION TAKEN | G | 18 | FUTURE ACTION | Z | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 22 | ATTACHMENT SUBMITTED | Y | 23 | NPRD-4 FORM SUB. | N | 24 | PRIME COMP. SUPPLIER | Z | 25 | COMPONENT MANUFACTURER | Z | 9 | 9 | 9 | 26 |
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The cause of this occurrence was due to procedural inadequacy. Since these valves are
11 | located in containment and could not be tested during normal operation, there was no
12 | guidance in procedure ST 5099.08 to test these valves in cold shutdown. Procedures
13 | ST 5099.08 and SP 1102.01 were modified to test these valves in cold shutdown,
14 | subsequent to the successful testing of these valves on August 12, 1981.
7 8 9 80

15 | FACILITY STATUS | E | 28 | % POWER | 0 | 9 | 9 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Discovered by NRC Resident Inspector | 32 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

16 | ACTIVITY RELEASED | Z | 33 | CONTENT OF RELEASE | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

17 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

18 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

19 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

20 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | NA | 45 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

8109280471 810916
PDR AD0CK 05000346
PDR

NRC USE ONLY

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TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-60

DATE OF EVENT: August 20, 1981

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: ASME Section XI testing requirements for containment isolation check valves CV 124, CV 125, NN 58, IA 501, and SA 502 were not met

Conditions Prior to Occurrence: The unit was in Mode 1 with Power (MWT) = 2755 and Load (Gross MWE) = 919

Description of Occurrence: During a maintenance outage in August, 1981, the NRC Resident Inspector found that containment isolation valves CV 124, CV 125, NN 58, SA 502, and SA 501 had not been tested as required per ASME Section XI, Inservice Inspection of Valves. ASME Section XI Article IWV-3422 states that valves not accessible for testing during normal operating, valve testing shall be on a frequency determined by the intervals between shutdowns as follows: for intervals of three months or longer, test the valves during each shutdown; for intervals of less than three months, full stroke exercise is not required unless three months have passed since last shutdown testing.

The valves were successfully tested per Surveillance Test ST 5099.08 on August 12, 1981. On August 20, 1981 at a Station Review Board review of a major modification for ST 5099.08, Miscellaneous Valve Quarterly Test, it was discovered that this occurrence is reportable per Technical Specification 4.0.5.

Designation of Apparent Cause of Occurrence: The cause of this occurrence is attributed to procedural inadequacy. The check valves are located in containment and not accessible during normal operation, and could not be tested quarterly when called for per the surveillance test schedule. There was no direction in either ST 5099.08, Miscellaneous Valves Quarterly Test or PP 1102.01, Prestartup Checklist, to test these valves in cold shutdown when required.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. These valves were tested per the Local Leak Rate Test during the 1980 refueling outage and all but CV 124 were found to be leaking and repaired, however, containment isolation was maintained by existing redundant isolation valves in series with the valves that were found inoperable. All these valves were successfully tested in cold shutdown on August 12, 1981.

Corrective Action: A new section has been added to ST 5099.08 per modification M-5169 which tests valves CV 124, CV 125, NN 58, SA 502, and IA 501, when in cold shutdown (Mode 5) if it has not been tested within the previous three months. A modification has been made to PP 1102.01 per modification T-5727 to ensure that during plant startup, the testing status of these valves is checked. The computerized surveillance test schedule has been modified to insure testing of these valves will be scheduled when the unit is in cold shutdown.

LER #81-050

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-60

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Failure Data: There has been a previous reportable occurrence concerning inservice testing of valves with a similar root cause, see Licensee Event Report NP-33-78-93 (78-078).

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