LICENSEE EVENT REPORT

CONTROL BLOCK:
0 1 N Y J A F 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 4 57 CAT 58
CON'T REPORT
0 12 A failure of management and administrative controls resulted in the replacement
of a safety relief valve in January, 1981 with a valve set at a different set point
then allowed by TS 2.2.1.B.
No significant hazard existed. See attachment for additional details.
07
08
SYSTEM CODE CAUSE SUBCODE COMPONENT CODE SUBCODE SUBCO
CODE TYPE NO. CODE TYPE TYPE NO. CODE TYPE TYPE TYPE NO. CODE TYPE T
Personnel error was the cause. TS was revised to allow interim operation with
valve of different set point. A valve with the original set point was installed
during the next shutdown. Additional action to prevent recurrence was taken.
See attachment for additional details.
1 4 6
PACILITY STATUS SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 THE STATUS NA LD 31 Resident Inspector THE STATUS AS SO STA
AGTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 NA NA NA NA NA NA NA NA NA N
FERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 1 7 8 9 11 12 13 13 13 13
PERSONNEL INJURIES NUMBER DESCRIPTION 41) NA NA
LOSS OF OR DAMAGE TO FACILITY 43
PUBLICITY ISSUED DESCRIPTION 45
NA NA S S S S S S S S S S S S S S S S S
108310444 810821 REPARER W. Verne Childs PHONE 315-342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 81-060/03L-0

Page 1 of 1

During a short plant outage from January 17, 1981 to January 25, 1981, Safety Relief Valve 02-RV-71G, with a set point of 1140 psig, was replaced with a spare valve having a set point of 1090 psig. Technical Specifications 2.2.1.B requires safety relief valves to be set as indicated below:

2 valves at 1090 psig (+ 1%) 2 valves at 1105 psig (+ 1%) 7 valves at 1140 psig (+ 1%)

Replacement of valve 02-RV-71G (set at 1140 psig), with a spare set at 1090 psig, resulted in safety relief valves set as indicated below:

3 valves at 1090 psig (+ i%)2 valves at 1105 psig (+ i%)6 valves at 1140 psig (+ i%)

On March 25, 1981 the NRC Resident Inspector noted the noncompliance which resulted in a request for interim plant peration with the 1090 psig valve in place of the 1140 psig valve. In addition, a number of other actions to prevent recurrence have been taken. Additional information is contained in Inspection Report 50-333/81-07 and Meeting Report 50-333/81-11.

NRC reporting requirements, with respect to this event, were discussed following identification of the noncompliance by the Resident Inspector and during plant staff meetings. These discussions resulted in the interpretation or understanding that a LER was not required until the event was again discussed with the Resident Inspector on August 10, 1981. Based on this most recent discussion, this LER is submitted as required by Technical Specifications 6.9.A.4.2.