

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun #1
 DATE August 10, 1981
 COMPLETED BY R. W. Short
 TELEPHONE (402)536-4543

MONTH July, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>437.3</u>	17	<u>432.4</u>
2	<u>437.8</u>	18	<u>432.5</u>
3	<u>437.7</u>	19	<u>431.9</u>
4	<u>439.2</u>	20	<u>431.4</u>
5	<u>438.9</u>	21	<u>433.4</u>
6	<u>437.0</u>	22	<u>433.2</u>
7	<u>434.1</u>	23	<u>434.0</u>
8	<u>434.0</u>	24	<u>435.6</u>
9	<u>433.9</u>	25	<u>433.8</u>
10	<u>435.4</u>	26	<u>436.0</u>
11	<u>435.0</u>	27	<u>439.1</u>
12	<u>433.1</u>	28	<u>442.0</u>
13	<u>430.7</u>	29	<u>441.9</u>
14	<u>429.5</u>	30	<u>441.4</u>
15	<u>429.8</u>	31	<u>442.1</u>
16	<u>431.8</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8108260117 810810
 PDR ADOCK 05000285
 R PDR

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE August 10, 1981
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 TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: July, 1981
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A

10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	5,087.0	68,808.0
12. Number Of Hours Reactor Was Critical	744.0	4,954.0	54,732.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	4,905.4	53,665.3
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,052,832.4	5,149,675.9	64,867,916.2
17. Gross Electrical Energy Generated (MWH)	340,586.0	1,687,060.0	21,475,331.6
18. Net Electrical Energy Generated (MWH)	323,903.5	1,582,943.5	20,281,126.4
19. Unit Service Factor	100.0	96.4	78.0
20. Unit Availability Factor	100.0	96.4	78.0
21. Unit Capacity Factor (Using MDC Net)	91.1	64.6	64.5
22. Unit Capacity Factor (Using DER Net)	91.1	64.6	64.1
23. Unit Forced Outage Rate	0.0	3.6	4.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling, September 15, 1981, 4-7 Weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1981

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE August 10, 1981
 COMPLETED BY R. W. Short
 TELEPHONE (402)536-4543

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									No outages during the month of July, 1981.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July 1981.

1. Scheduled date for next refueling shutdown. September 15, 1981
2. Scheduled date for restart following refueling. November 15, 1981
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No
- a. If answer is yes, what, in general, will these be?
- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. No
- c. If no such review has taken place, when is it scheduled? August 1, 1981
4. Scheduled date(s) for submitting proposed licensing action and support information. _____
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. _____
6. The number of fuel assemblies:
- | | | |
|---|------------|------------|
| a) in the core | <u>133</u> | assemblies |
| b) in the spent fuel pool | <u>197</u> | " |
| c) spent fuel pool storage capacity | <u>483</u> | " |
| d) planned spent fuel pool storage capacity | <u>483</u> | " |
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

J R L

Date

August 3, 1981

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

July 1981
Monthly Operations Report

I. OPERATIONS SUMMARY

The unit operated at 95% Reactor Power during the month of July.

Preparations for the SRO examination continued. The exams were administered by the NRC on July 29, 1981.

Surveillance Tests and Operational Tests were performed.

No Safety Valve or PORV challenges occurred on the primary system.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
LER 81-006	<p>The "B" channel Nuclear Power Recorder decreased approximately 30% in power level (as opposed to the remaining 3 channels). "B" channel T_{HOT} decreased to approximately 507°F from the normal operating temperature of 588°F. The ΔT power level dropped approximately 15% from its normal reading (and the readings of the remaining three channels). In addition, "B" Subcooled Margin Monitor went into alarm. The reactor was operating at approximately 94% power at the time. The RPS was subsequently placed in a two-out-of-three logic to trip modes (i.e., "B" channel was placed in bypass.</p> <p>It was discovered that an inadvertent ground in the NI recorder and one in a temperature loop for that channel of the RPS had caused a current loop which provided the path for the incongruous readings. The grounds were isolated and the calibrations of the temperature loop and recorder checked to verify proper operation. The RPS was returned to full operability.</p>
LER 81-007	<p>During routine power operations at approximately 95% power, control room operators received several indications that diesel generator DG-1 had tried to start. These indications included:</p> <ol style="list-style-type: none">(1) the "engine start" light being lit,(2) the diesel air dampers going open,(3) the "engine running" computer signal, and(4) the "diesel trouble" annunciator. Immediate inspection revealed that diesel D1 had not actually started.

A. PERFORMANCE CHARACTERISTICS (Continued)

LER 81-007 Diesel generator DG-2 was satisfactorily tested for operability and along with the 345 kv and 161 kv feeds (which were also available) more than adequate power was available to mitigate the consequences of an accident if necessary.

It was discovered that the speed sensing module was defective and gave the erroneous indication. The module was replaced with a calibrated module and the Diesel returned to service.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incident</u>		<u>Deficiency</u>
OI-1347	ST-RM-3	Calibration of Process Radiation Monitoring. (1) Facility license change number 7801 corrects this technical specification discrepancy and is still awaiting NRC Action. (2) Completion of calibration (3) Completion of Surveillance Test Revision.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel Assembly Uplift Condition Detection the two operable Δp channels shows greater than 99% assurance that the 150 pound criterion is not being violated. An unreviewed safety question as defined in 10CFR50.59 did not exist as this procedure only involves evaluating data from a surveillance test.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION
APPROVAL (Continued)

Procedure

Description

EEAR FC-78-49

Battery Room Ventilation completed as designed.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 since it was made to reduce the chances of fire in the rooms in which the Station batteries are contained.

EEAR FC-79-54

Water Plant Supply Pump Modification completed as designed.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because the work done was on a non-safety related system.

EEAR FC-80-94

Vacuum Priming Trap Replacement completed as designed.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because the work done was on a non-safety related system.

DCR FC-75B-7

Raw Water Pressure Switches completed as designed.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because it only involved installing voltage suppressors across switches to prevent induced DC transients.

DCR FC-75A-6

Auxiliary Building Hot Standby/Minor Piping Modification in piping run.

DCR 75A-6 involved piping modifications to the auxiliary boiler to reduce thermal shock encountered when the boiler is placed in operation and to improve pressure regulation.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because the work done was on a non-safety related system.

DCR FC-77-104

Expansion of Computer Room completed as designed.

This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because it only involved expansion of the computer room. Necessary security plan requirements were taken during the modification.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

<u>Procedure</u>	<u>Description</u>
EEAR FC-79-218	Jockey Pump completed as designed. This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because Tech. Specs. limits on outage time for the fire protection system were not exceeded.
EEAR FC-78-39	Installation of drain lines and isolation valves in the plant Compressed Air System. This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because the work done provided for better reliability of the compressed air system.
EEAR FC-78-50A	HVAC for Switchgear Rooms completed as designed. This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because it only involved air conditioning the Switchgear Rooms to help improve equipment reliability.
EEAR FC-78-50	Switchgear Room Separation completed as designed. This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because it only involved dividing the Switchgear Room with fire barriers to help ensure a fire would not take out redundant safety related equipment.
DCR FC-76-82	Installation of Unions on boration system safety valves. This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 because it only involved installation of appropriate Nuclear Class fittings to remove relief valves for inservice inspection as required by Tech. Specs.

E. RESULTS OF LEAK RATE TESTS

On 7-29-81 PAL door tested at 60 psig. Leak rate = 0.0 sccm

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

Training for July included:

General Employee training and retraining for security badges.

Maintenance training on systems and Tech. Specs.


Operator requalification training program.

SRO training for 3 candidates to sit for NRC exam in late July.

Safety meetings for each craft.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY
COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

NONE

Approved By 
Manager - Fort Calhoun Station

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
10093	7-14-81	Area Monitor RM-088 spiking into alert.	Cleaned switch and contacts.
10019	7-14-81	AI-41B breaker and cable faulted.	Replaced cable.
10241	7-14-81	VOPT Channel C reset demand push-button on CB/does not reset.	Replaced switch.
10243	7-14-81	Air Sampler has high flow.	Replaced filter and cartridge.
10648	6-25-81	RPS "B" Safety Channel T _{HOT} Swing.	Found grounds at 2 points in the current loop B/122-H. Repaired
10404	6-8-81	"A" Raw Water Strainer Breaker stops and doesn't trip.	Freed Strainer.
10410	6-4-81	Containment Particulate Monitor RM-050 Take-up Motor faulted.	Replaced Take-up Motor.
10272	5-22-81	RM-076 Radiation Monitor Set-point out of tolerance.	Cleaned switch alarm contacts.
10240	6-10-81	AI-137 Improper indication for smoke damper override switch 83A & 83B off normal.	Replaced fuses on power supplies and rewired.
10231	5-20-81	Charging Pump CH-1B trips free when trying to start.	Replaced control switch.
10841	7-10-81	Aux. FW Auto Initiation LI-906Y-1 Setpoints not within tolerance.	Adjusted agastat and retested.
10595	6-3-81	Noise in Charging Pump CH-1B gear Box.	Realigned and replaced broken line.
10871	7-14-81	Fire Pump FP-1B running 70 RPM low.	Reset RPM at 1760.
10903	7-21-81	Charging Pump CH-1B tank hot overflowing.	Repaired per MP-CH-1.
10903	7-16-81	Heat Tracing TAR-2 Point 12 indicating >300°F.	Cleaned selector relay & contactor.