

# Advanced Structural Materials for Non-Light Water Reactors

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**Workshop on Advanced Non-Light Water Reactors: Materials and  
Component Integrity**

**Nuclear Regulatory Commission**

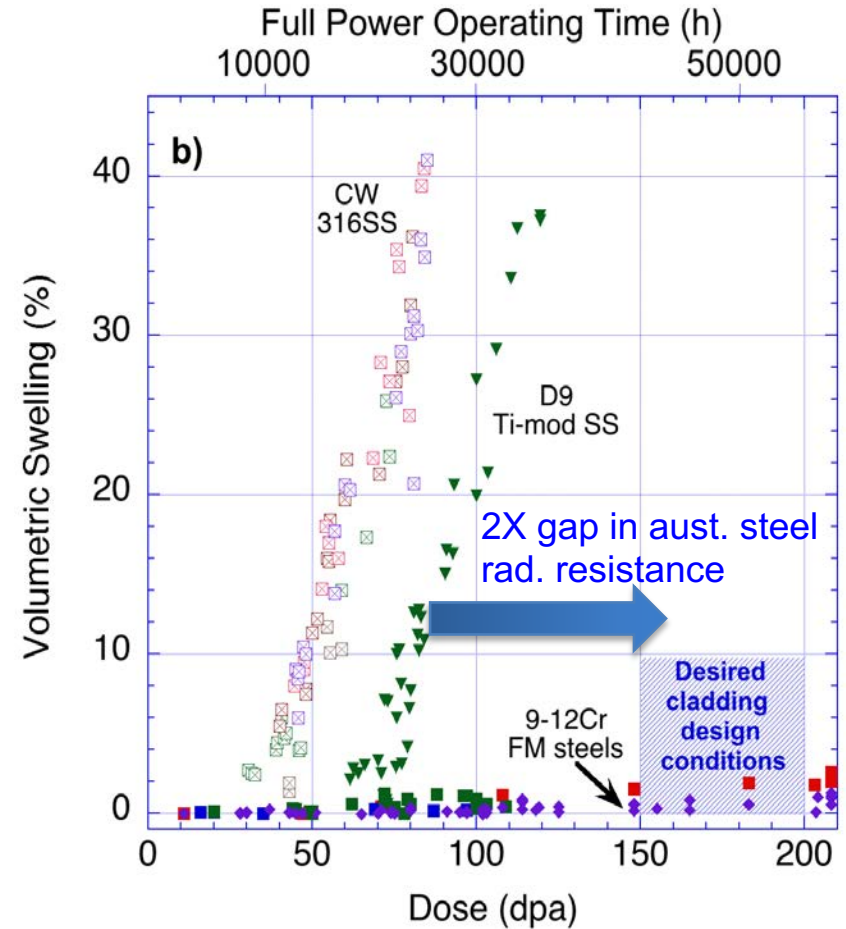
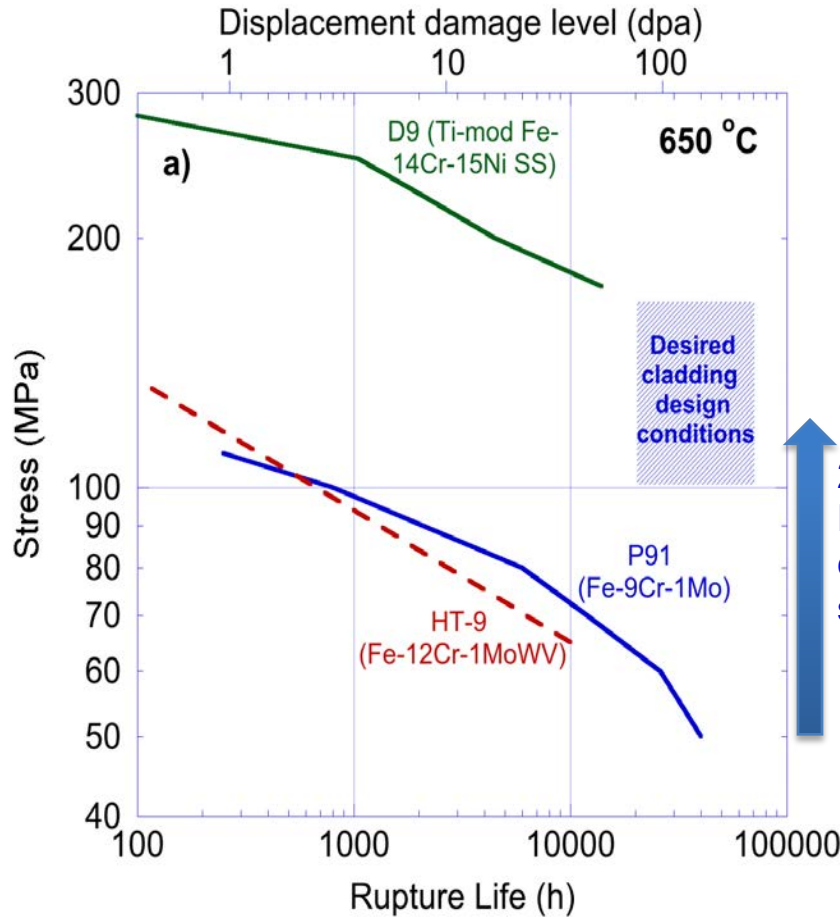
**December 9-11, 2019**

# Overview

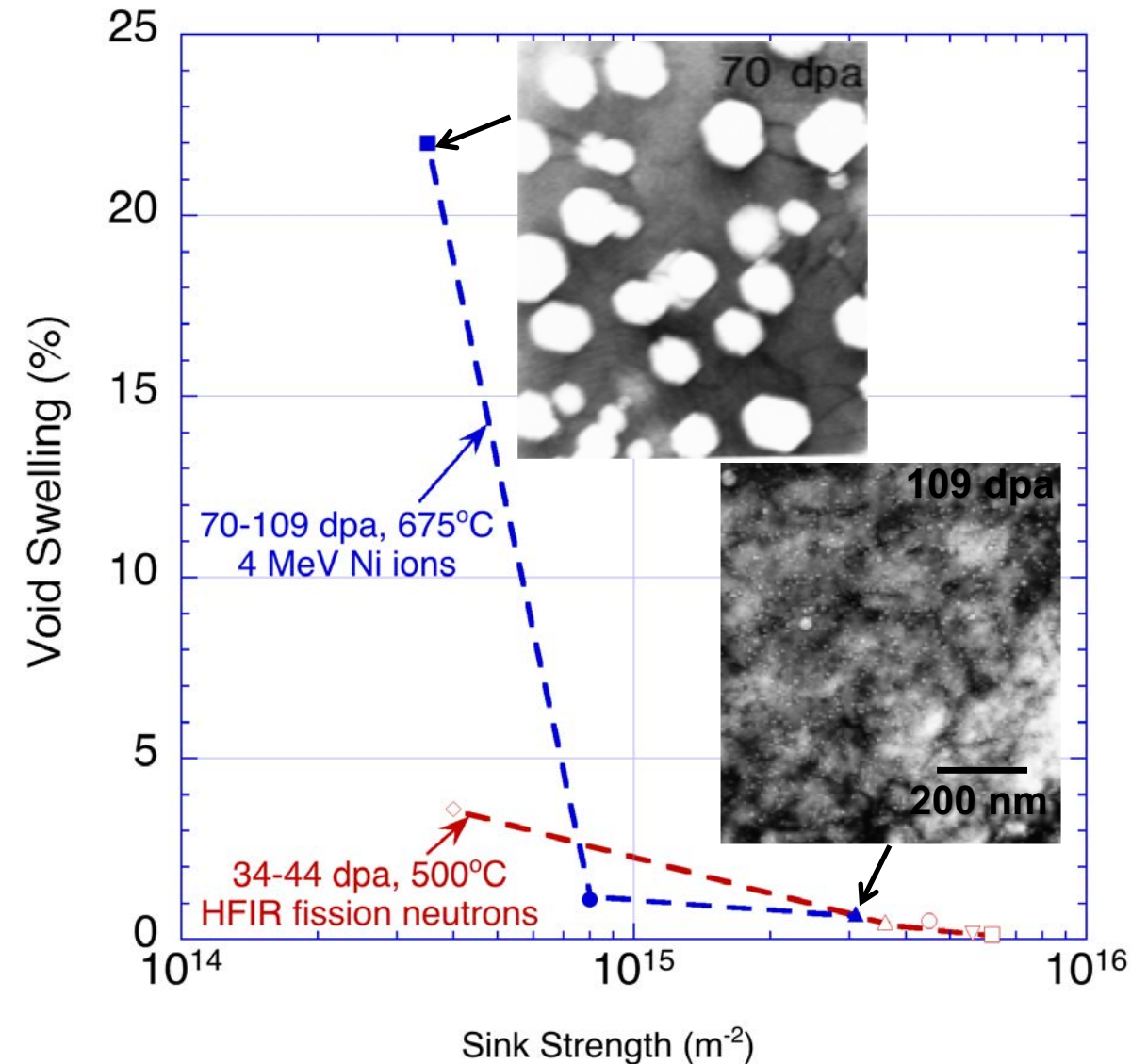
- High sink strength has been a long-standing scientific tenet for superior radiation resistance in structural alloys
  - Cold-worked and Ti-modified SS alloys (e.g., D9) developed by LMFBR program in the 1970s
- Improved structural materials are needed for nuclear power to fully achieve its promise
  - High burnup, accident tolerant LWRs
  - Fusion and Gen IV reactors
- Nanostructured alloys enable simultaneous achievement of radiation resistance and high performance (strength)
  - Radiation resistance (“sink strength”):  $S \sim 4\pi RN$
  - Dispersed barrier hardening:  $\Delta\sigma_y \sim \alpha\mu b(2NR)^{1/2}$
  - ⇒ For a given precipitate volume  $f = 4\pi R^3/3$ , best radiation resistance and mechanical strength simultaneously occurs for high N, small R
    - ↖  $S \sim N^{2/3}/R^2$
    - ↖  $\Delta\sigma_y \sim f^{1/2}/R$
  - ⇒ High density of uniformly distributed nanoscale precipitates preferred

# Thermal creep and void swelling in sodium-cooled fast reactor cladding is problematic for conventional steels

Current “either/or” dilemma if structural alloy selection is limited to conventional steels



# Effect of initial sink strength on the volumetric void swelling of irradiated FeCrNi austenitic alloys

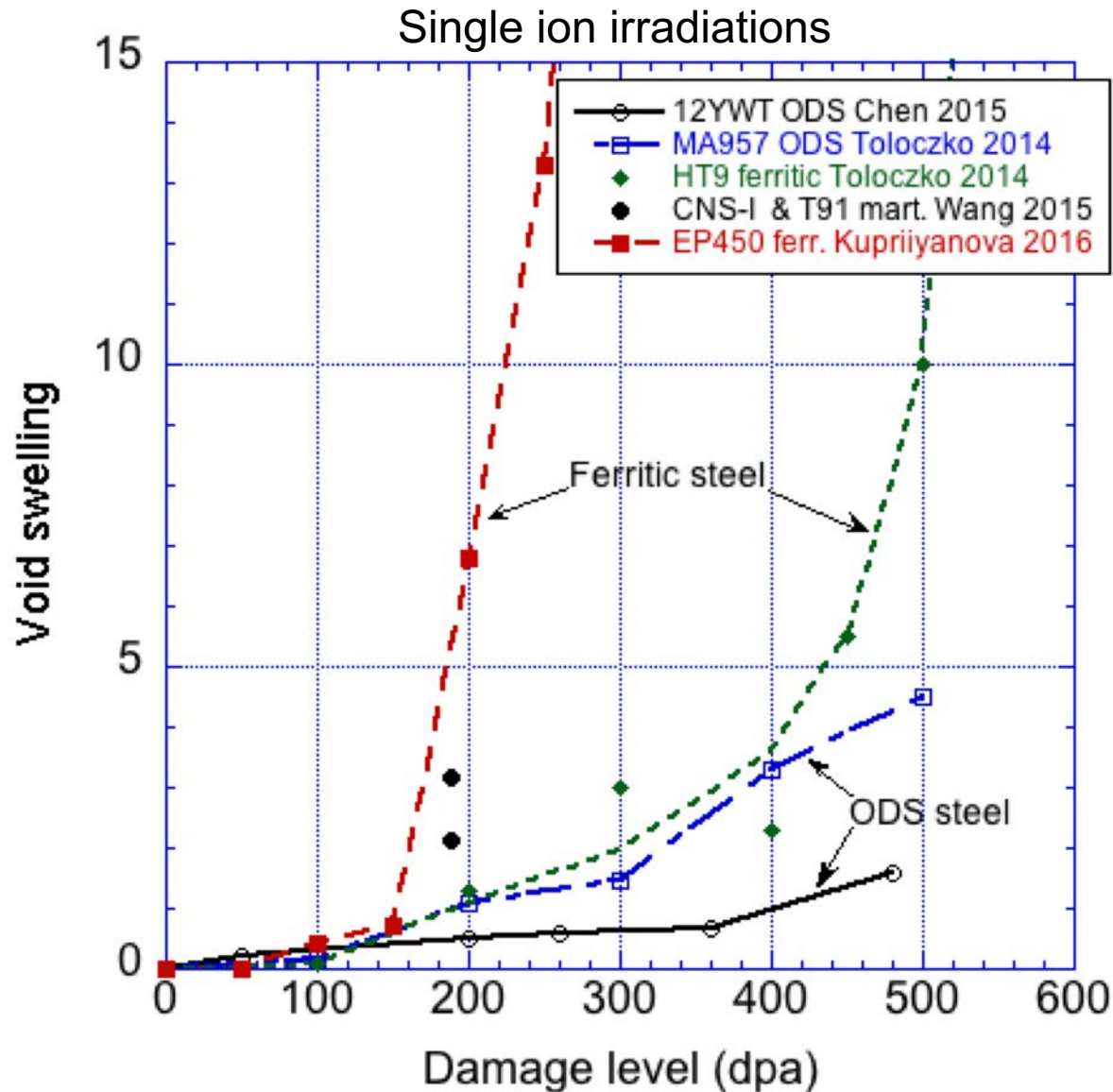


Dramatic reduction in void swelling occurs when average spacing between voids is  $>10\times$  average spacing between defect sinks

$$N_v^{-1/3} > 10 S_{\text{tot}}^{-1/2}$$

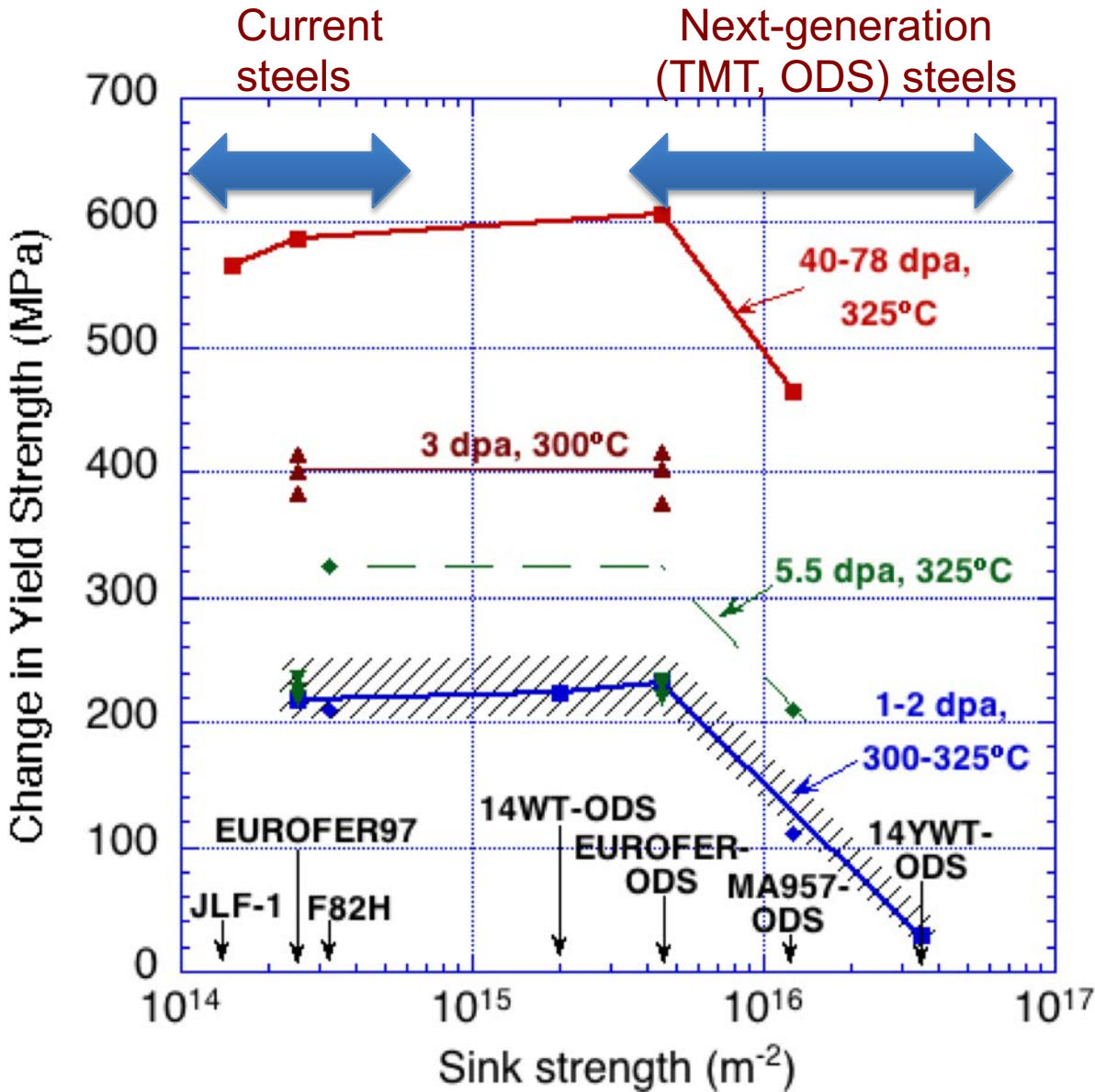
For void swelling resistance, sink strengths  $>10^{15}/\text{m}^2$  are generally sufficient for fission reactors; fusion reactor irradiation may require even higher sink strengths ( $>10^{16}/\text{m}^2$ ?) due to transmutant He production

# ODS steels provide improved void swelling resistance compared to standard ferritic/martensitic steels





# Effect of Initial Sink Strength on the Radiation Hardening of Ferritic/martensitic Steels



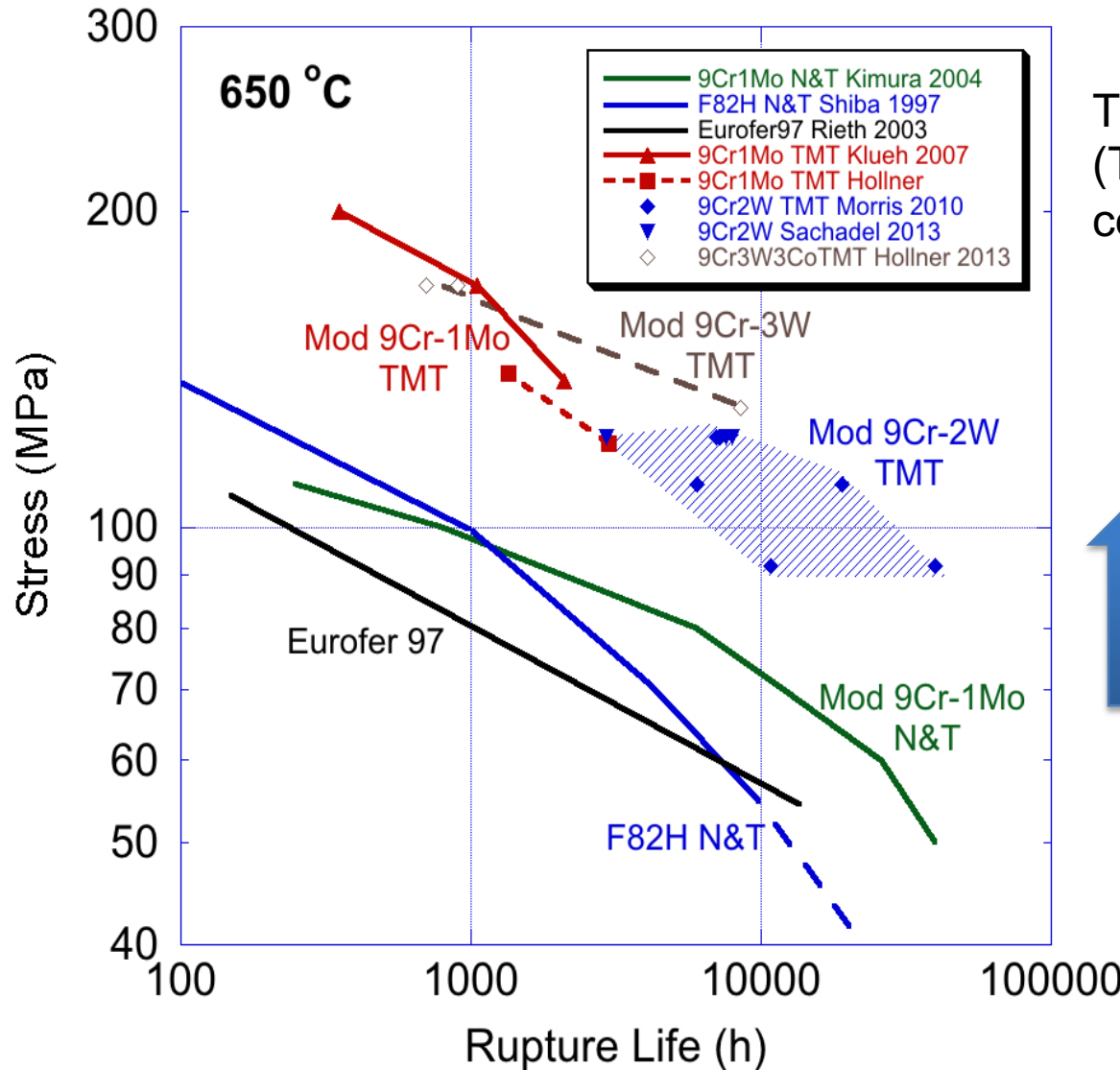
Dramatic reduction in radiation hardening occurs when average spacing between defect cluster nuclei (dislocation loops, etc.) is much greater than average spacing between defect sinks

$$N_{loop}^{-1/3} \gg S_{tot}^{-1/2}$$

or equivalently,

$$S_{tot} \gg S_{rad defects}$$

# Creep rupture behavior for TMT vs. conventional 9Cr steels

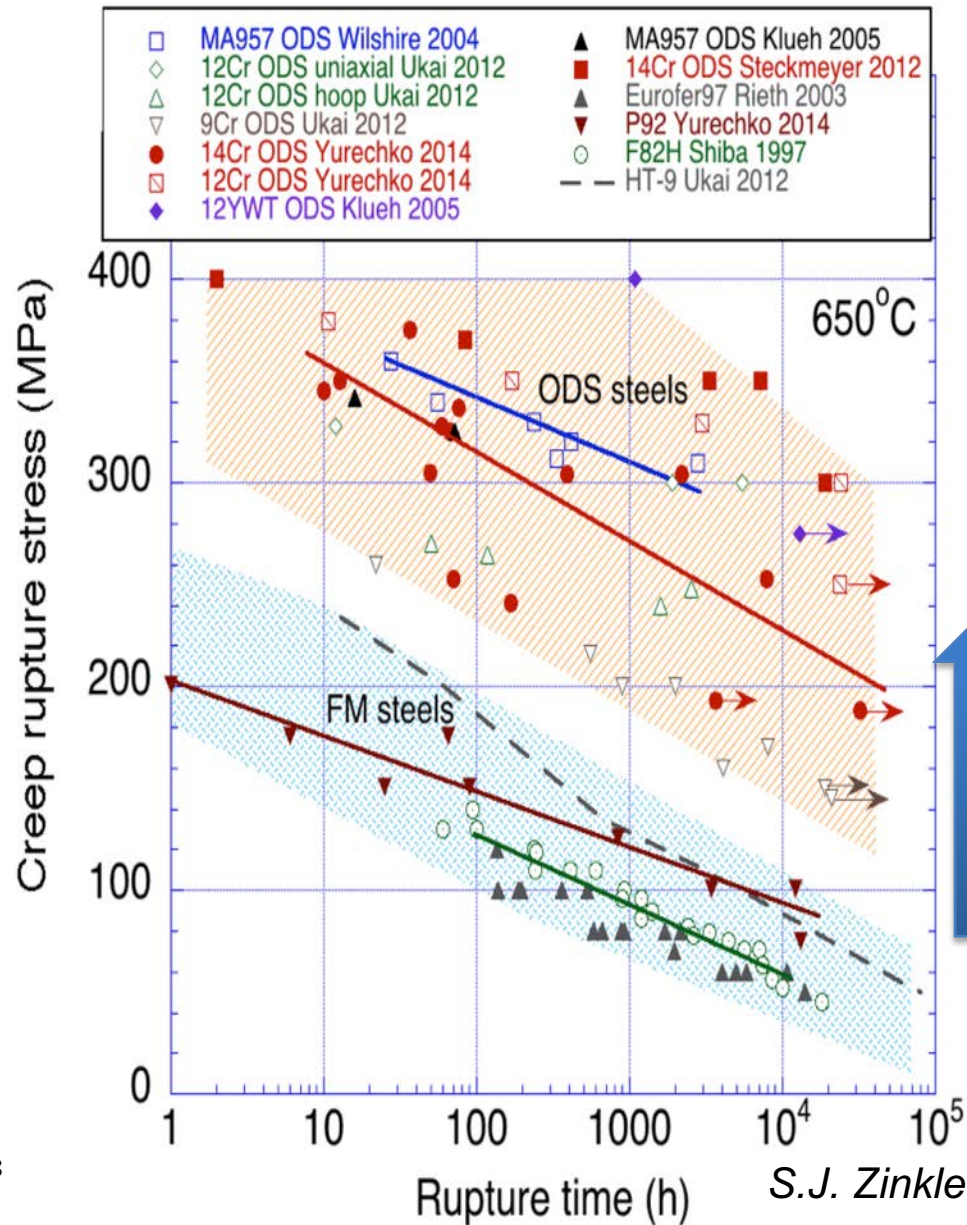


Thermo-mechanical treatment (TMT) 9Cr steels designed using computational thermodynamics

50-100% improvement in creep rupture strength for newly designed reduced activation steels

Predicted improvement in radiation resistance as well due to high precipitate density

# Creep rupture behavior for ODS vs. conventional 9Cr steels



~100% improvement in creep rupture strength for ODS steels

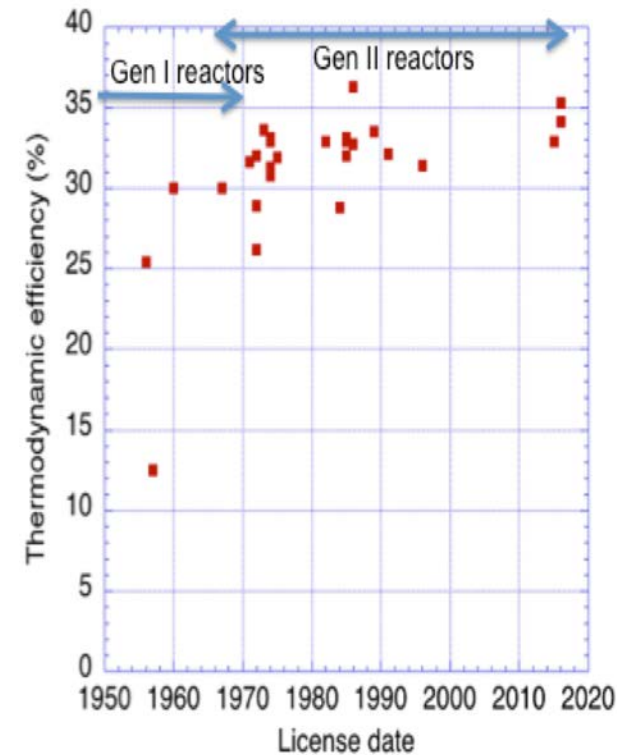
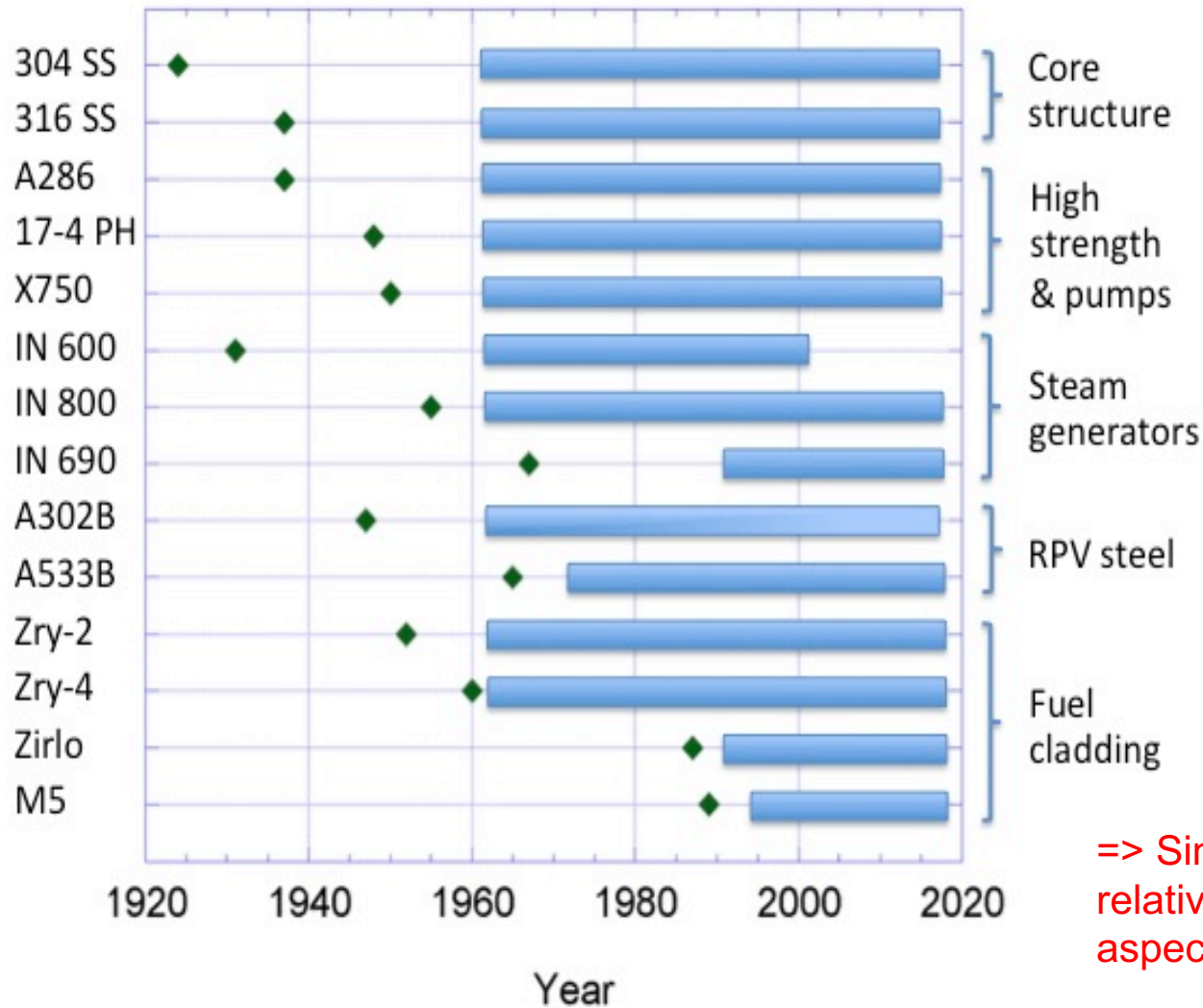
Predicted improvement in radiation resistance as well due to high dispersoid density



# Conclusions

- Nanostructured (high sink strength) alloys are promising options for the structural materials in next-generation fission reactors
  - Enables simultaneous superior radiation resistance and superior mechanical property performance
- ASME code qualification is needed to enable their deployment
  - Currently in “boutique” materials stage

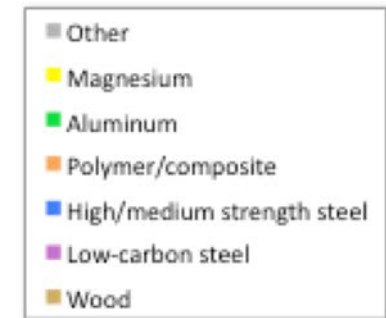
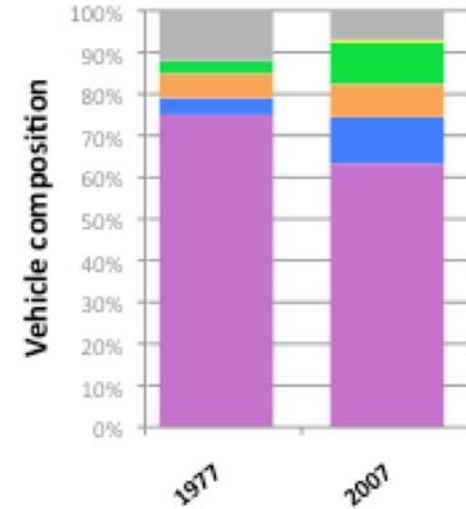
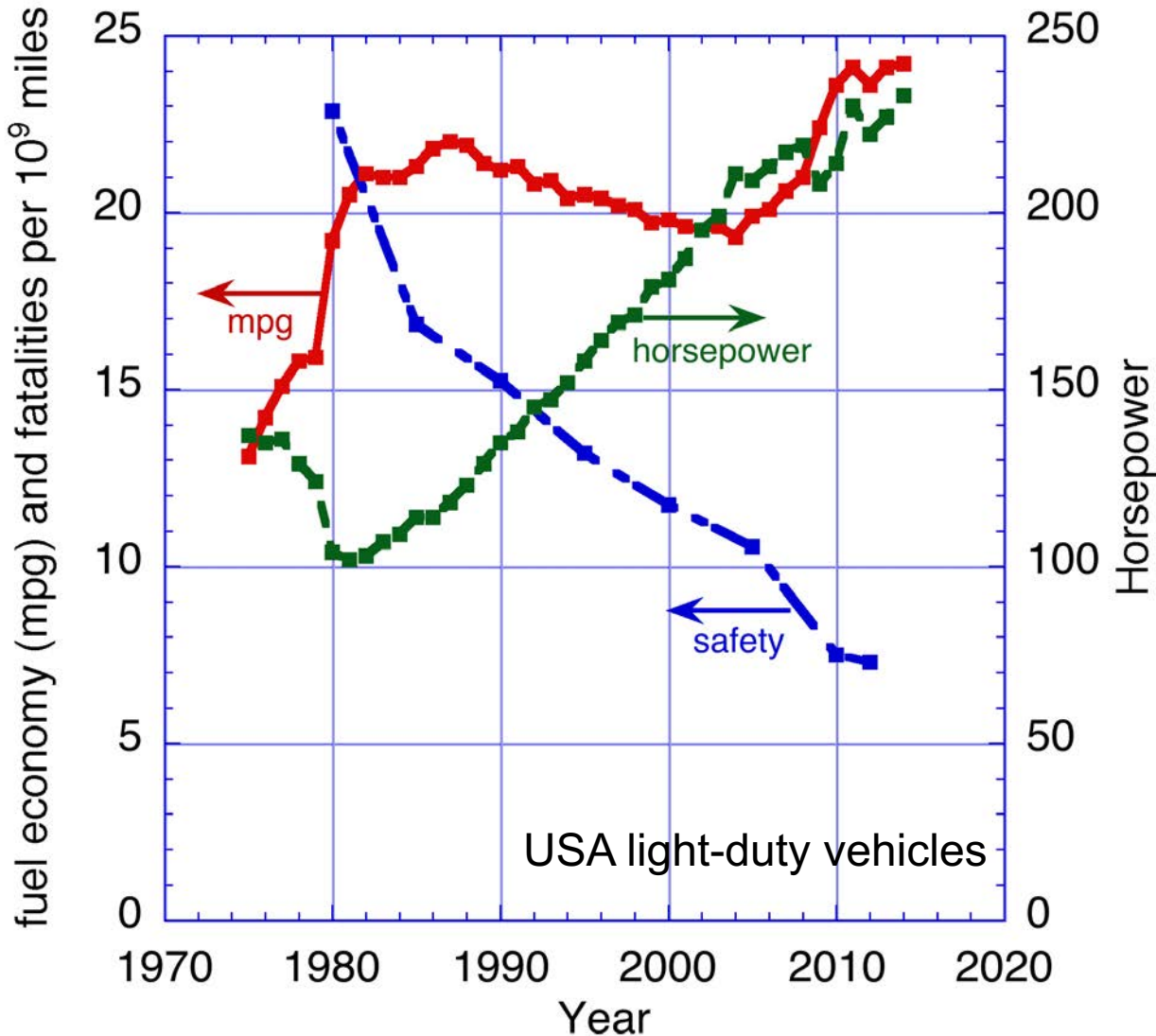
# Timeline of structural materials used in light water reactors



=> Since the 1960s, there has been relatively little modification of materials aspects for LWRs

# Evolution in light duty personal vehicles

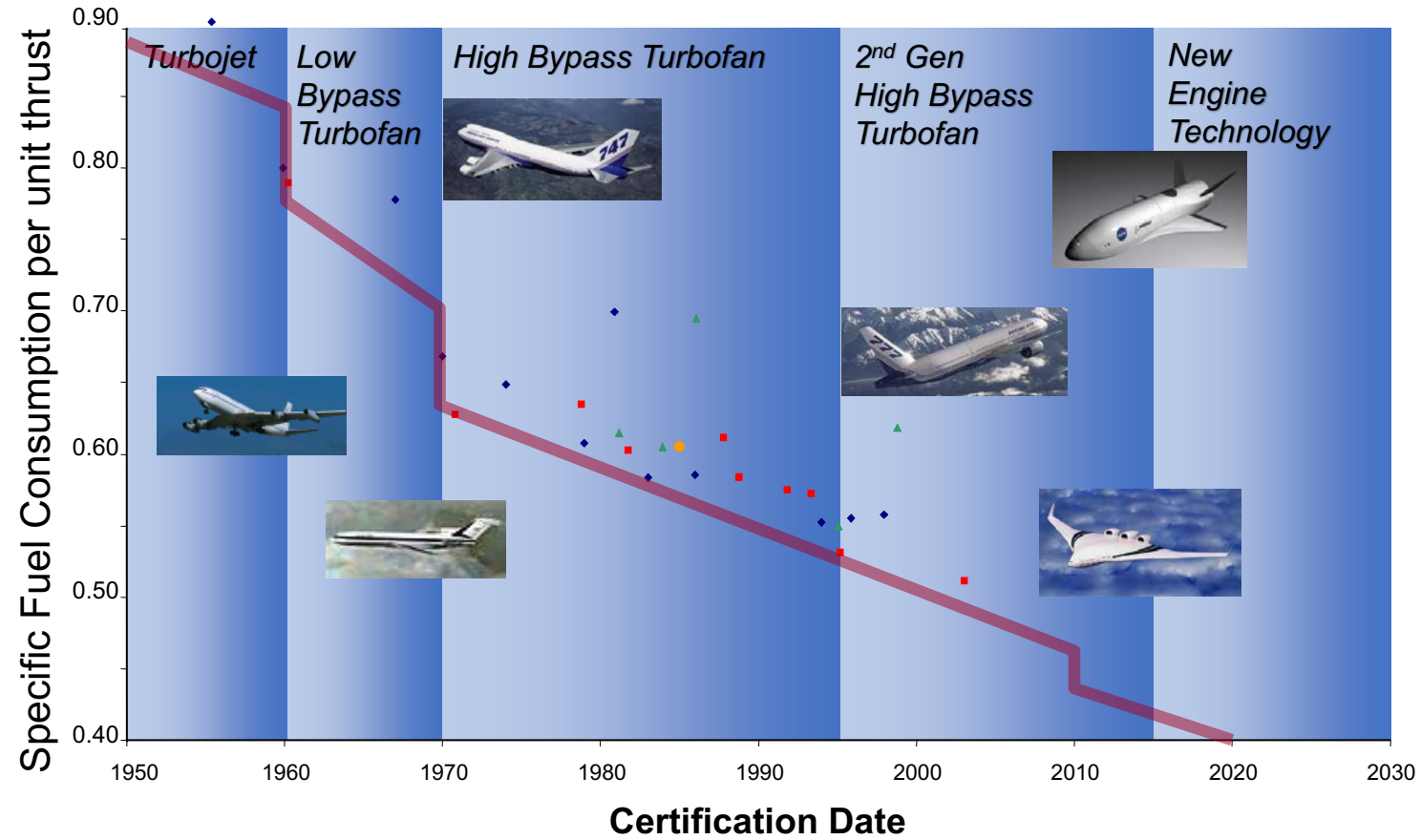
Greatly improved safety, along with improved performance (horsepower, fuel economy)



A. Taub et al., JOM 59, 2 (2007) 48

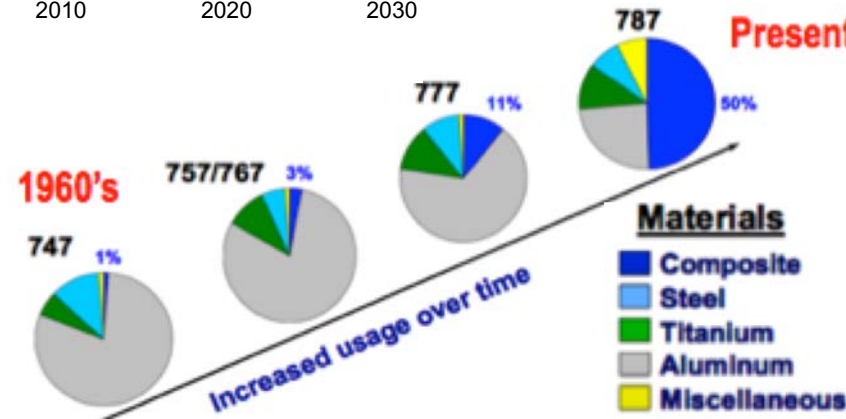
# New Propulsion Materials and Architectures Have Driven Marked Improvements in Jet Engine Fuel Efficiency

Plus: 40x improvement in durability; 10x improvement in in-flight shutdown rate



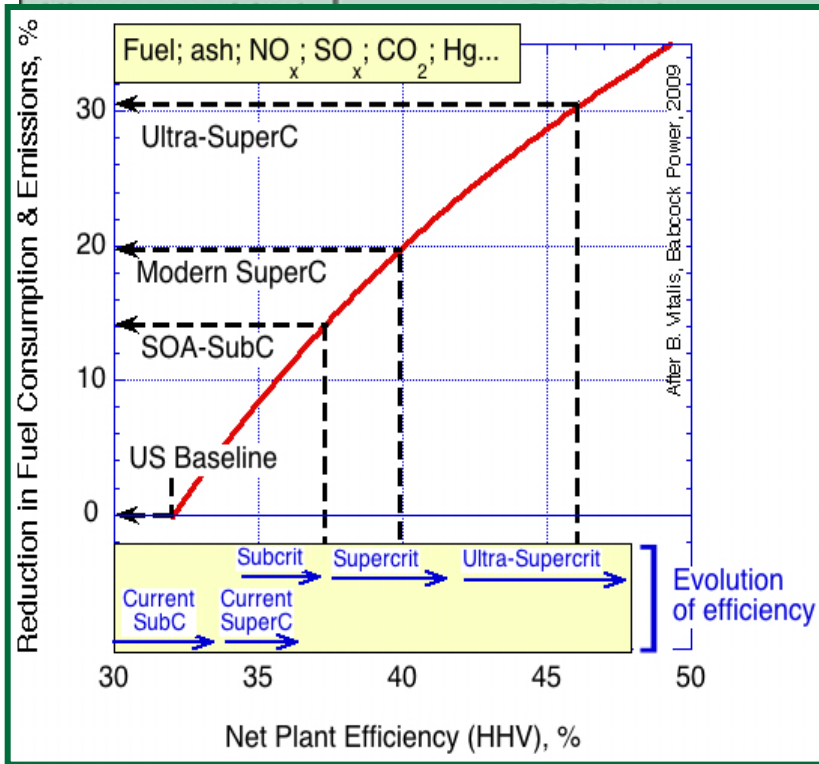
R.E. Schafrik, personal communication

W.G. Roeseler et al., in 16th Int. Conf. on Composite Materials, Kyoto, Japan (2007)



# Near-term Importance of Advanced Steam

Nomenclature	Conditions	Net plant efficiency (%)	Net plant heat rate (HHV)
Subcritical	2,400 psi	35	9,751 Btu/kWh
	1,050F/1,050F		
Supercritical	>3,600 psi	38	8,981 Btu/kWh
	1,050F/1,075F		
		>42	8,126 Btu/kWh
		>45	7,757 Btu/kWh

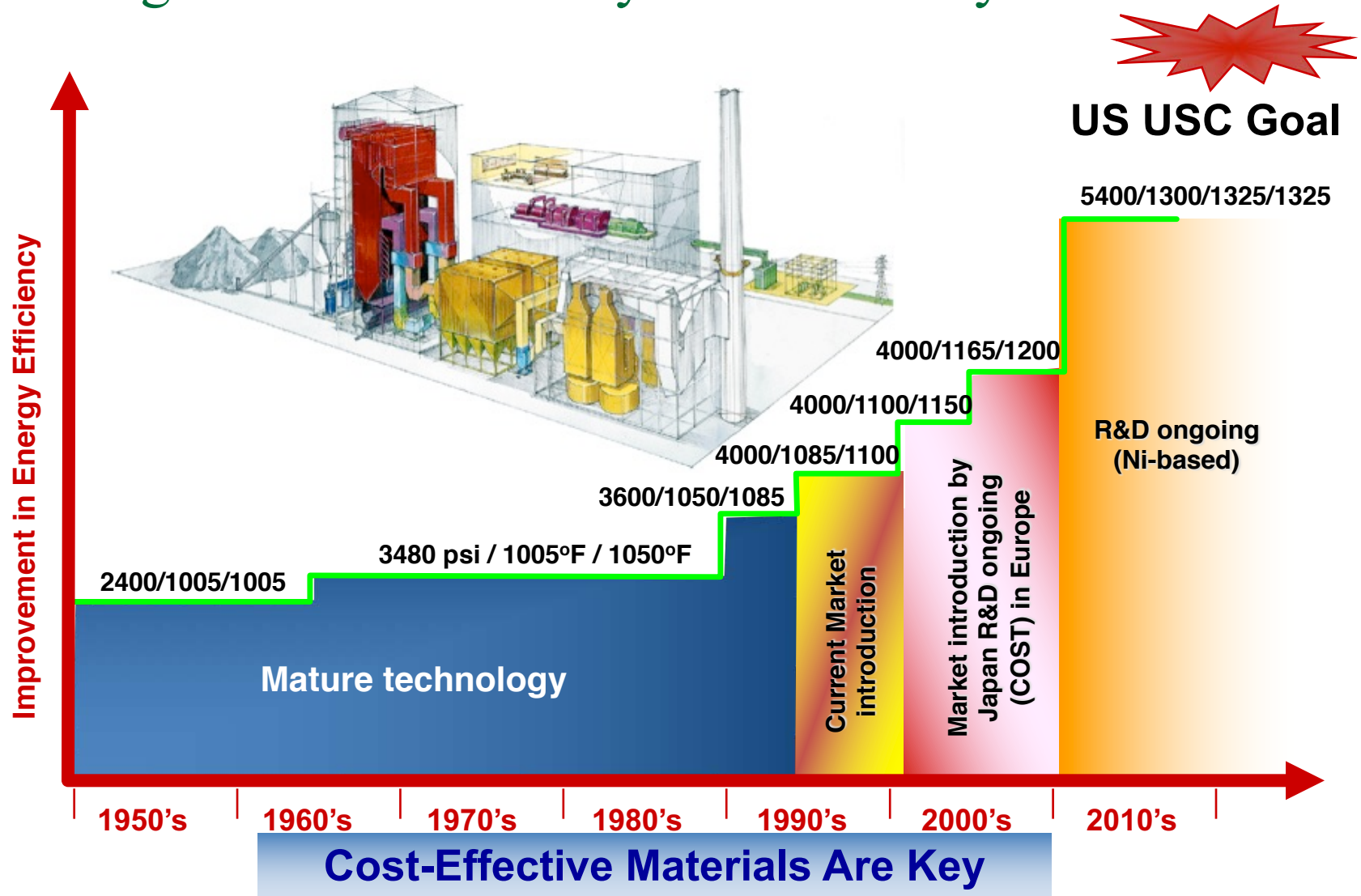


Source: Electric Power Research Institute

essential to reduce cost of CCS

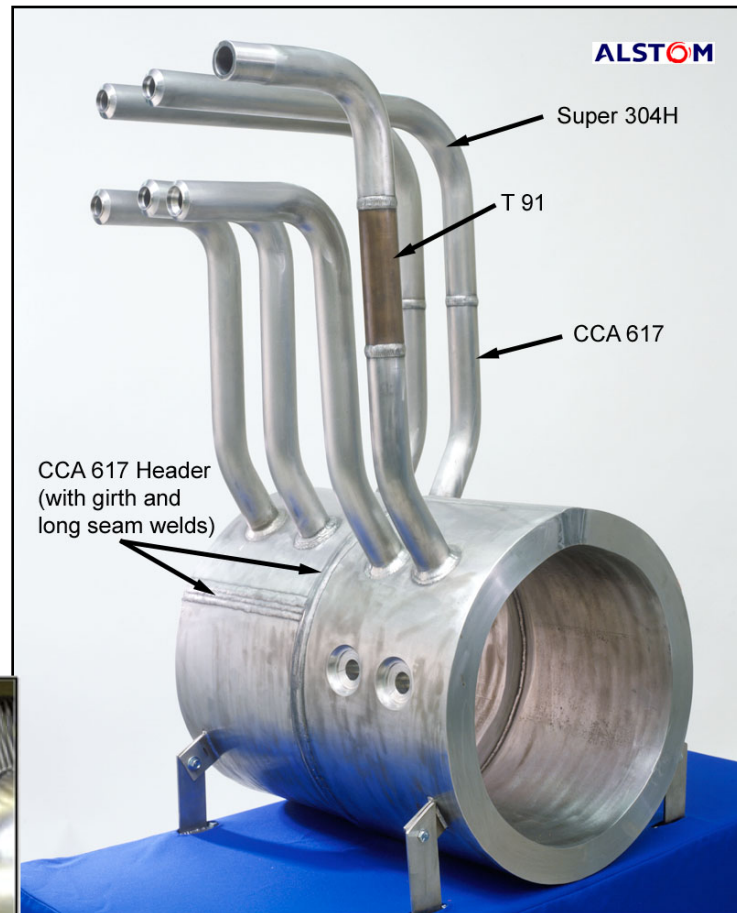
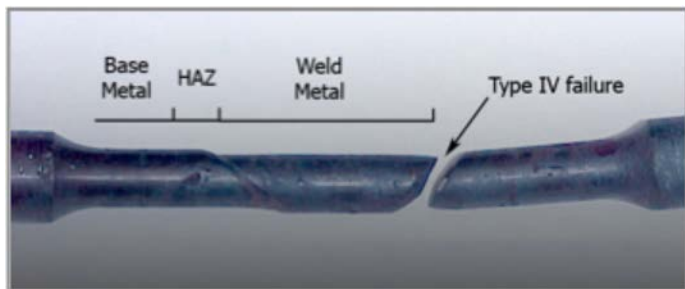


# Advanced Steam Conditions Push Metallurgical Limits of Alloys in Use Today



# Tubing/piping materials for advanced ultra-supercritical (A-USC) steam (760°C, 350 bar)

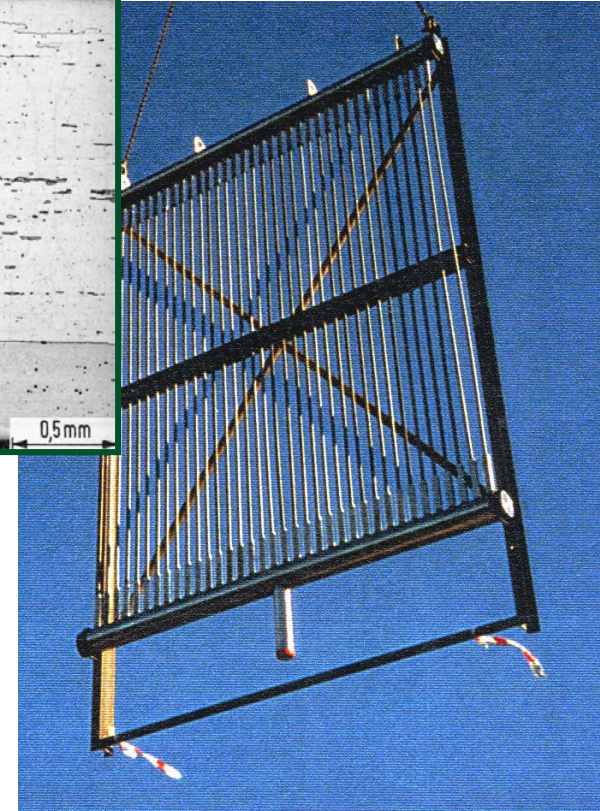
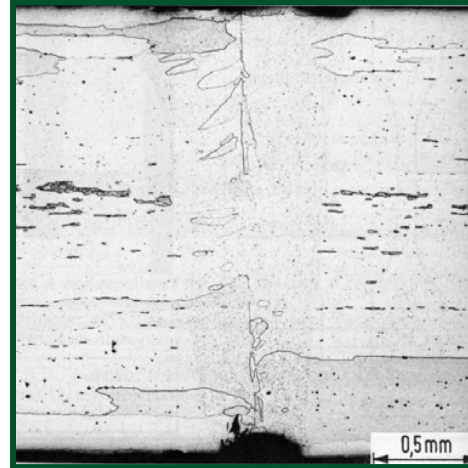
- Objective: qualify/develop advanced alloys to enable reliable, high efficiency operation of A-USC plants
- Key Deliverables:
  - Generate alloy properties database for U.S. boiler manufacturers to enable component design and fabrication
  - Qualification of fabrication/welding techniques for use with specific alloys
  - ASME Boiler and Pressure Vessel Code case for Inconel 740
  - Evaluate environmental compatibility



Demonstration header fabricated from advanced alloys qualified by the U.S. A-USC Consortium

# Qualification of new, commercial ODS alloys for use in advanced FE processes

- Objective: Determine capabilities of new commercially-produced ODS alloys for application at temperatures up to 1200C in advanced fossil combustion and conversion processes



25 mm diam x 4 m long ODS FeCrAl tubes used in British Gas/COST 501 1100°C air heater demonstrator

- Key Deliverables
  - Data on the full range of properties required to qualify the alloy for fossil applications
  - Evaluate joining technologies
  - Feasibility of employing a less costly route for producing ODS alloy powders



# Historical development of improved high-temperature steels has exhibited slow and steady progress

Based on R. Viswanathan, Adv. Mat. Proc. 162 (2004) 73

