



**Entergy  
Operations**

**Entergy Operations, Inc.**  
P.O. Box B  
Kilona, LA 70066  
Tel 504-464-3120

**D. F. Packer**  
General Manager  
Plant Operations  
Waterford 3

**W3F1-94-0050**  
**A4.05**  
**PR**

May 13, 1994

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Subject: Waterford 3 SES  
Docket No. 50-382  
License No. NPF-38  
Reporting of Licensee Event Report

Gentlemen:

Attached is Licensee Event Report Number LER-94-005-00 for Waterford Steam Electric Station Unit 3. This Licensee Event Report is submitted in accordance with 10CFR50.73(a)(2)(i).

Very truly yours,

D.F. Packer  
General Manager  
Plant Operations

DFP/GCS/tjs  
Attachment

cc: L.J. Callan, NRC Region IV  
G.L. Florreich  
J.T. Wheelock - INPO Records Center  
R.B. McGehee  
N.S. Reynolds  
NRC Resident Inspectors Office (WADM526)  
Administrator - LRPD

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FACILITY NAME (1) **Waterford Steam Electric Station Unit 3** DOCKET NUMBER (2) **05000** PAGE (3) **1 OF 5**

TITLE (4) **Valves in the Inservice Test Plan - Pump and Valve, not Tested**

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	15	94	94	005	00	05	13	94	n/a	05000
									n/a	05000

OPERATING MODE (9) <b>5</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
POWER LEVEL (10) <b>0</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME **John Houghtaling, Mgr. Tech. Services** TELEPHONE NUMBER (include Area Code) **504/464-3131**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 15, 1994, a Shift Technical Advisor discovered that several valves were not being tested per the requirements of ASME section XI. Specifically valves SI-6011, SI-6012, CVR-401A and CVR-401B remote position indicators were not being observed at least once every two years to verify that valve position matched remote indication. Also identified was that power operated valves SBV-113A, SBV-113B, SBV-114A and SBV-114B were not being timed in their closed stroke direction. Technical Specification 4.0.5(a) requires that inservice testing of ASME Code Class 1, 2, and 3 pumps and valves be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code.

The root cause of this event is inappropriate action resulting in a failure to properly incorporate IST Plan requirements into the appropriate operating procedures during initial IST Plan implementation. A Pump & Valve Review Team has been established to perform an in depth review of the Plan and the associated implementing documents. This event did not compromise the health and safety of the public.

REQUIRED NUMBER OF DIGITS/CHARACTERS  
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME  8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

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8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
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12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
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14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Waterford Steam Electric Station Unit 3	05000 382	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 5
		94	- 005	- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**REPORTABLE OCCURRENCE**

On April 15, 1994, a Shift Technical Advisor discovered that valves were not being tested per the requirements of ASME section XI. Specifically valves SI-6011 (EIIS Identifier BP-FSV), SI-6012 (EIIS Identifier BP-FSV), CVR-401A (EIIS Identifier BF-FSV) and CVR-401B (EIIS Identifier BF-FSV) remote position indicators were not being observed at least once every two years to verify that their valve operators were accurately indicated as per the requirements of ASME Section XI Article IWV-3300. Also power operated valves SBV-113A (EIIS Identifier VC-20), SBV-113B (EIIS Identifier VC-20), SBV-114A (EIIS Identifier VC-20) and SBV-114B (EIIS Identifier VC-20) were not being timed in their closed stroke direction as per the requirements of ASME Section XI Article IWV-3413(b). Technical Specification 4.0.5(a) requires that inservice testing of ASME Code Class 1, 2, and 3 pumps and valves be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. The failure to test these valves per ASME section XI requirements is a violation of Waterford 3 Technical Specification 4.0.5(a) which requires testing of ASME code class 1, 2, and 3 pumps and valves. Per 10CFR50.73(a)(2)(i)(B) any operation or condition prohibited by the plant's Technical Specification is reportable as an LER.

**INITIAL CONDITIONS**

Plant Power	0%
Plant Operating Mode:	Mode 5; Cold Shutdown
Procedures Being Performed Specific to This Event:	None
Technical Specification LCO's in Effect Specific to this Event:	None
Major Equipment Out of Service Specific to this Event:	None

**LICENSEE EVENT REPORT (LER)  
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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Waterford Steam Electric Station Unit 3	05000 382	94	005	00	3 OF 5

TEXT (If more space is required, use additional copies of NRC Form 365A) (17)

**EVENT SEQUENCE**

On April 15, 1994 a Shift Technical Advisor (STA) discovered during a review of the Waterford 3 Pump and Valve Inservice Test Plan (IST) that valves SI-6011, SI-6012, CVR-401A, CVR-401B, SBV-113A, SBV-113B, SBV-114A and SBV-114B were in the IST plan as required but their testing requirements were only partially addressed in the plant's implementing procedures.

Valves SI-6011, SI-6012, CVR-401A and CVR-401B have been in the IST Plan since revision 1 dated April 4, 1984. The ASME Section XI requirement to verify that valve position indicators are accurately indicating has never been proceduralized and consequently never verified. Valves SBV-113A, SBV-113B, SBV-114A and SBV-114B have been in the IST Plan since revision 0 dated August 16, 1982. The ASME Section XI requirement to time these valves in the closed stroke direction has also never been proceduralized and consequently never verified.

**CAUSAL FACTORS**

The root cause of this event is inappropriate action resulting in a failure to properly incorporate IST Plan requirements into the appropriate operating procedures during initial IST Plan implementation. Although past corrective actions associated with the IST problems appeared to be adequate, it is believed that due to the accumulation of unrelated concerns identified with the Plan, a full review of it and its implementing documents is warranted.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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Waterford Steam Electric Station Unit 3	05000 382	94	-- 005	-- 00	3 OF 5

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**EVENT SEQUENCE**

On April 15, 1994 a Shift Technical Advisor (STA) discovered during a review of the Waterford 3 Pump and Valve Inservice Test Plan (IST) that valves SI-6011, SI-6012, CVR-401A, CVR-401B, SBV-113A, SBV-113B, SBV-114A and SBV-114B were in the IST plan as required but their testing requirements were only partially addressed in the plant's implementing procedures.

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Waterford Steam Electric Station Unit 3	05000 382	94	-- 005	-- 00	4 OF 5

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**IMMEDIATE CORRECTIVE MEASURES**

The required IST testing of valves SI-6011, SI-6012, CVR-401A, and CVR-401B was performed on April 17, 1994 per work authorization WA 01123100. A procedure change for OP-903-043 was initiated to time the closure of valves SBV-113A, SBV-113B, SBV-114A and SBV-114B per the requirement of the IST plan.

**ACTIONS TO PREVENT RECURRENCE**

A Pump & Valve Review Team has been established to perform an in depth review of the IST Plan to ensure all current IST requirements are procedurally implemented.

A periodic test method will be implemented for valves SI-6011, SI-6012, CVR-401A and CVR-401B to verify valve position indications.

**SAFETY SIGNIFICANCE**

The purpose of the inservice testing of the valves identified in this LER was to determine the amount of valve degradation. Upon the discovery that valves SI-6011, SI-6012, CVR-401A, and CVR-401B position indicators were not being verified they were immediately tested and proven to be operational.

Although plant procedures have no time requirement associated with stroking these valves closed, valves SBV-113A, SBV-113B, SBV-114A, and SBV-114B were stroked in both the open and close position per plant procedure OP-903-043



**LICENSEE EVENT REPORT (LER)  
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" Shield Building Ventilation System Operability Check" . As such, the operability of the valves was demonstrated.

Based on the above, this event did not compromise the health and safety of the general public or plant personnel.

**SIMILAR EVENTS**

Problems with the IST Plan were identified in LER 89-014 and LER 90-010. In LER 89-014 the Auxiliary Component Cooling Water valves (EIIS Identifier BI-V) ACC-116A and ACC-116B were not included in the Pump & Valve IST program. The root cause of this event was misinterpretation of the ASME Section XI definition of 'active ' valves. Because ACC-116A and ACC-116B were judged to be 'passive' in relation to the ASME XI definition , they were never included in the IST program. In LER 90-010 feedwater check valves, FW-181A and FW-181B, and both Emergency Diesel Generator fuel oil transfer pumps had been excluded from the Pump & Valve IST program. The root cause of this event was an inadequate review of ASME Section XI IST requirements during the development of the Pump & Valve IST Plan program. Although these events are related to the events described in this LER, the significant difference is that for the events of this LER there were no discrepancies identified with the Pump and Valve IST plan but with the procedural implementation of the plan.