

OPPD

Omaha Public Power District
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Omaha, Nebraska 68102-2247
402/636-2000

May 13, 1994
LIC-94-0093

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: April 1994 Monthly Operating Report (MOR)

Enclosed is the April 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/mah

Enclosures

- c: LeBoeuf, Lamb, Greene & MacRae
- L. J. Callan, NRC Regional Administrator, Region IV
- S. D. Bloom, NRC Project Manager
- R. P. Mullikin, NRC Senior Resident Inspector
- R. T. Pearce, Combustion Engineering
- R. J. Simon, Westinghouse
- Office of Management & Program Analysis (2)
- INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

APRIL 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of April, the station operated at a nominal 100% power level. The spent fuel pool reracking project continued.

On April 4, a DC Bus #2 ground alarm was received causing numerous toxic gas monitor alarms and erratic Qualified Safety Parameter Display System (QSPDS)/Core Exit Thermocouple indications. Investigation indicated that the alarm was related to the #2 Battery Charger input to QSPDS Channel B. A lead was lifted and the ground was eliminated. A faulty capacitor was subsequently found and the channel was declared inoperable. The capacitor was replaced and QSPDS Channel B was declared operable on April 8.

A problem was discovered with the closing spring for the circuit breaker for Raw Water Pump AC-10D. The problem was corrected by replacing the closing spring. Previous experience within the industry has revealed failures in the springs as a breaker of this type approaches 2000 cycles. The Electric Driven Fire Pump FP-1A was subsequently declared inoperable due to the concern that its breaker closing spring was approaching 2000 cycles. It was declared operable on April 12 after replacement of its closing spring. Other components were inspected and in some cases the springs were replaced, but none of the other components were close to the 2000 cycle threshold.

On April 18, an Electro-Hydraulic Control (EHC) system problem occurred. With no action on the part of the operating crew, turbine control valves moved slightly, evidenced by a small shift in the Reactor Coolant System (RCS) cold-leg temperature. An EHC circuit card is suspected of causing the problem and plans are in progress to replace the card. The EHC control cabinet is now open to improve ventilation. The condition is being closely monitored.

On April 23, Omaha Public Power District reported an inadvertent release of about 1500 pounds of sulfuric acid to a berm which drains to the neutralization basin. Although not a direct release to the environment, it exceeded State of Nebraska release limits and was consequently reported. A pump failing to shut down properly caused the inadvertent release.

On April 28, personnel discovered Waste Disposal Pump WD-23A to be smoking and glowing red after startup to recirculate Monitor Tank WD-22A. The Auxiliary Building Operator secured the pump and applied CO₂ from a fire extinguisher to the pump motor. Abnormal Operating Procedure AOP-06, "Fire Emergency," was entered and subsequently exited.

The room was ventilated to disperse the smoke. It was discovered that WD-23A had no suction path because the outlet valve for WD-22D was shut; therefore, causing the pump to overheat.

The following NRC inspections were completed during this reporting period:

IER No. Description

- 94-11 Emergency Plan Walkdown Inspection
- 94-13 Solid Radwaste and Transportation Programs

The following Licensee Event Reports were submitted during this reporting period:

LER No. Description

- 94-002 Inoperability of Boric Acid Pump Due to Inappropriate Feeder Breaker
- 94-003 Inoperability of Raw Water Pumps Due to Excessive Sand Accumulation

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of April, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

RCS leak rate was steady throughout the month of April. The leak rate was a nominal 0.10 gpm, with no degrading trends noted. The only changes observed were due to normal plant transients and periodic temporary increases from charging pump packing leaks.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. Description

- 162 The amendment makes changes to the Technical Specifications to revise the minimum requirement of fuel oil that must be in the Emergency Diesel Generator Fuel Oil Storage Tank.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE

- Installed new CR120A relays (74-1-1/A3, 1B3A, 1B3B and 1B3C) on DC Sequencer SI-1 Circuit Alarms 3, 4, 5, and 6 and new CR120A relays (74-1-2/1A3, 1B3A, 1B3B and 1B3C) on AC Sequencer SI-2 circuit alarms 3, 4, 5, and 6.
- Replaced a circuit board assembly and a faulty capacitor in Circuit Board DT-701-2 on QSPDS Channel B.

- Replaced Linear Amplifier Cards A1 and A2 for the Power Range Safety Drawer (AI-31C-CW3-A1).
- Rebuilt Charging Pump CH-1A.
- Replaced a coupling on Charging Pump CH-1B.
- Installed bushings on Raw Water Outlet Valve Operators HCV-2880B-0 and HCV-2883B-0 for Component Cooling Water Heat Exchangers AC-1A and AC-1D.
- Replaced the closing spring for the Circuit Breaker 1A4-12 for Raw Water Pump AC-10D.
- Installed closing springs of a new design in the following breaker units: 1A3-10/Raw Water Pump AC-10C, 1A3-16/Auxiliary Feedwater Pump FW-6, 1A3-7/Low Pressure Safety Injection (LPSI) Pump SI-1A, 1A4-11/Raw Water Pump AC-10B, 1A1-0/Fire Pump FP-1A, 1A3-20/Diesel Generator DG-1, 1A4-14/LPSI Pump SI-1B.
- Installed Furmanite boxes to seal Swagelok fitting leak on Main Steam Bypass Valve HCV-1042C.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
 UNIT FORT CALHOUN STATION
 DATE MAY 09, 1994
 COMPLETED BY M. A. HOWMAN
 TELEPHONE 402-533-6939

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
 2. Reporting Period: APRIL 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
 4. Nameplate Rating (Gross MWe): 502
 5. Design Elec. Rating (Net MWe): 478
 6. Max. Dep. Capacity (Gross MWe): 502
 7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
 N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
 N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	719.0	2879.0	180553.0
12. Number of Hours Reactor was Critical	719.0	2845.2	140536.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	719.0	2830.1	138893.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1076021.3	4125311.7	183411111.3
17. Gross Elec. Energy Generated (MWH)..	364978.0	1398892.0	60486916.2
18. Net Elec. Energy Generated (MWH)....	348641.7	1335331.5	57708394.4
19. Unit Service Factor.....	100.0	98.3	76.9
20. Unit Availability Factor.....	100.0	98.3	76.9
21. Unit Capacity Factor (using MDC Net)	101.4	97.0	69.3
22. Unit Capacity Factor (using DER Net)	101.4	97.0	67.6
23. Unit Forced Outage Rate.....	.0	1.7	4.2

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
 NONE

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FOPT CALHOUN STATION
DATE MAY 09, 1994
COMPLETED BY M. A. HOWMAN
TELEPHONE 402-533-6939

MONTH APRIL 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	487	17	485
2	487	18	484
3	486	19	483
4	486	20	482
5	487	21	483
6	487	22	483
7	487	23	484
8	487	24	483
9	487	25	481
10	486	26	481
11	486	27	481
12	486	28	483
13	486	29	485
14	487	30	485
15	486	31	N/A
16	486		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE May 9, 1994
COMPLETED BY M. A. Howman
TELEPHONE (402) 533-6939

REPORT MONTH April 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None									During April 1994, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending April 30, 1994

1. Scheduled date for next refueling shutdown. March 11, 1995
2. Scheduled date for restart following refueling. April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? No
 - a. If answer is yes, what, in general, will these be? N/A
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. No
 - c. If no such review has taken place, when is it scheduled? Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information. No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. **
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 570 Assemblies
 - c) spent fuel pool storage capacity 729 Assemblies
 - d) planned spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1995 Outage*

* Capability of full core offload of 133 assemblies lost. Reracking began in March and is scheduled for completion in August 1994.

** OPPD is planning to utilize CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by Kenn Heltman Date 5-9-94