

NOTICE OF VIOLATION

Centerior Service Company
Perry Nuclear Power Plant

Docket No. 50-440
License No. NPF-58

As a result of the inspection conducted on January 30 through March 9, 1994, and in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, (Enforcement Policy) the following violations were identified:

1. 10 CFR Part 50, Appendix B, Criterion V, requires, in part, that activities affecting quality be prescribed by and accomplished in accordance with, documented instructions, procedures, or drawings of a type appropriate to the circumstances. These instructions, procedures, or drawings shall include appropriate quantitative or qualitative acceptance criteria for determining that important activities have been satisfactorily accomplished.
 - a. Contrary to the above, on February 2, 1994, after a water hammer occurred, the shift supervisor failed to log the occurrence of the water hammer in the plant log and failed to inform plant management, as required by Procedure PAP-0201, "Conduct of Operations".
 - b. Contrary to the above, Surveillance Test Procedures SVI-B21-T9124, T9416, T9122, and T9415, and Periodic Test Instructions PTI-B21-P001, 2, 3, and 4, used on February 14 and 15, 1994, during leak rate testing of the main steam lines were inappropriate to the circumstances in that they failed to contain a qualitative acceptance criteria for instrument air for the main steam isolation valve actuators.
 - c. Contrary to the above, on February 25 and 26, 1994, surveillance testing of safety relief valve (SRV) logic was not accomplished in accordance with procedures appropriate to the circumstance. Surveillance Procedures SVI B21-T0370A, "SRV Pressure Actuation Channel Calibration for 1B21-NO68A and 1B21-NO68E," and SVI B21-T0370B, "SRV Pressure Actuation Channel Calibration for 1B21-NO68B and 1B21-NO68F" both had an error which unintentionally caused an SRV to open. Also, on February 25, 1994, during the conduct of the SVIs, the Lead Test Performer did not stop the test when unexpected plant behavior was experienced, as required by Procedure PAP-1105, "Surveillance Test Control".
 - d. Contrary to the above, on March 1, 1994, during work on Emergency Diesel Generator 1R43C0001A (1A), the measurement of piston ring gaps was not accomplished in accordance with the documented instructions in work order (WO) 94-763 in that the measurements were not taken as directed by step 5.3.2 and Attachment 2 of work instruction PMI-0053, "Division 1 and 2 Standby Diesel Generator Connecting Rod and Piston Maintenance".

This is a Severity Level IV violation (Supplement I).

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2. Appendix B of 10 CFR Part 50, Criterion II, requires in part, that activities affecting quality shall be accomplished under suitably controlled conditions. Controlled conditions include the use of appropriate equipment, suitable environmental conditions for accomplishing the activity, such as adequate cleanliness, and assurance that all prerequisites for the given activity have been satisfied.

Contrary to the above, adequate cleanliness was not maintained for the Division 1 emergency diesel generator during corrective maintenance, as evidenced by a rag observed inside following the corrective maintenance.

This is a Severity Level IV violation (Supplement I).

Dated at Lisle, Illinois
this 25th day of April 1994