

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.5.3 Inspection Frequencies - The above-required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months, but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months* after the previous inspection. If two consecutive inspections following service under all volatile treatment (AVT) conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category, or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the inservice inspection of a steam generator, conducted in accordance with Table 4.4-2 and/or Table 4.4-6 requires a third sample inspection whose results fall in Category C-3, the inspection frequency shall be reduced to at least once per 20 months. The reduction in inspection frequency shall apply until a subsequent inspection demonstrates that a third sample inspection is not required.
- c. Additional unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 and/or Table 4.4-6 during the shutdown subsequent to any of the following conditions:
 1. Primary-to-secondary tube leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2,
 2. A seismic occurrence greater than the Operating Basis Earthquake,
 3. A loss-of-coolant accident requiring actuation of the engineered safeguards, or
 4. A main steam line or feedwater line break.

4.4.5.4 Acceptance Criteria

- a. As used in this Specification:
 1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings

* The twenty four months inspection interval scheduled to end by May 29, 1992, may be extended until Refuel 8.

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SHOLLY EVALUATION OF REQUEST:

Florida Power Corporation (FPC) has reviewed the requirements of 10CFR50.92 as they relate to the proposed change of the eddy current testing surveillance interval requirements. FPC considers the proposed one time change not to involve a significant hazards consideration. In support of this conclusion the following analysis is provided:

1. The proposed change will not significantly increase the probability or consequences of an accident previously evaluated because analysis of previous inspection data does not indicate or predict that tube failures are more likely to occur as a result of an extension to the inspection window. The consequences of a tube rupture accident are not affected by this change since the frequency and magnitude of steam generator tube leaks are not predicted to increase. Reactor coolant system leak detection and recovery actions remain unaffected by this change.
2. The proposed change will not create the possibility of a new or different kind of accident from any accident previously evaluated because the change will not alter plant configuration or change parameters governing normal plant operation. The FSAR steam generator tube rupture analysis continues to bound the conditions for steam generator tube failures.
3. The proposed change will not involve a significant reduction to the margin of safety because the magnitude and frequency of steam generator tube failures are not significantly affected by this change. The primary to secondary system leakage monitoring system and criteria remain unchanged. Recovery actions are unaffected by this change and sufficient to ensure a safe plant shutdown and any off-site releases are maintained within the FSAR accident analysis guidelines.