



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

February 27, 1991

Prairie Island Technical
Specification TS6.7A 5&6

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Prairie Island Nuclear Generating Plant
Docket No. 50-282 License No. DPR-42
Docket No. 50-306 License No. DPR-60

Effluent and Waste Disposal Semiannual Report
for July 1, 1990 through December 31, 1990

In accordance with the Prairie Island Technical Specifications, Appendix A to Operating License DPR-42 and DPR-60, we are submitting one copy of the Effluent and Waste Disposal Semiannual Report, covering the last half year of 1990.

Analysis for isotopes Sr-89 and Sr-90 for the 4th quarter were not completed in time to be included with this report. These analyses results are not expected to significantly change the computed off-site doses, and will be included with the next semiannual report.

Enclosed with this report is an amendment to the previously submitted Effluent and Waste Disposal Semiannual Report for the first half year of 1990. This revision includes the 2nd quarter composite results for Sr-89, Sr-90 and Fe-55. This revision also includes corrections to nuclide totals and doses for the 2nd quarter of revision 0. Revision 0 incorrectly used weeks 13 through 26 for the 2nd quarter calculations instead of weeks 14 through 26. This error caused week 13 to be included in both the 1st and 2nd quarters.

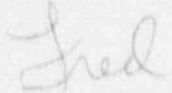
Revision 3 of the Process Control Program (PCP) has also been included with this report. The change occurred prior to the last reporting period, however it was not included in the last report. In this revision, a new section was added to describe the processing of certain liquid wastes through spent bead resin. These liquids include mop bucket slurry, tank bottoms, sump bottoms or other sources not suitable for evaporation or ion-exchange. The PCP changes have been side-barred.

9103040270 910227
PDR ADOCK 05000282
R PDR

LEAG
11

Revised pages of revision 11 of the Off-Site Dose Calculation Manual (ODCM) are being resubmitted to include a detailed description of the changes.

Additionally, there was one broad leaf sample which was lost, either in transit or in the off-site lab, during the reporting period.


F. L. Fey, Jr., Manager
Nuclear Radiological Services

Attachment